

# 중성자 측정 표준과 중성자 표준장

2016 년 10월 26일

박 현 서



# 발표내용

- 중성자 측정 표준 및 중성자장
- KRISS 연구내용
  - 동위원소 선원
  - 열중성자 선원
  - Workplace field
  - 단색에너지 중성자 선원 [가속기 발생]
- 향후 계획

# Neutron 측정 표준의 역할

- **Neutron field for calibration and test**
  - Well defined neutron fields
  - Energy distribution of neutrons
  - Flux (or fluence) of neutrons
- **Neutron measurement for the characterization of the neutron field**

# Standard Neutron field

## ➤ ISO 8529 Reference neutron radiations

### Reference Radionuclide neutron sources for calibrating neutron-measuring devices

Source	$\tau_{1/2}$	$E_n$ (MeV)	$h_\phi(10)$ pSv*cm <sup>2</sup>
<sup>252</sup> Cf(D <sub>2</sub> O moderated)	2.65	0.55	105
<sup>252</sup> Cf	2.65	2.13	385
<sup>241</sup> Am-B( $\alpha$ ,n)	432	2.72	408
<sup>241</sup> Am-Be( $\alpha$ ,n)	432	4.16	391

# Standard Neutron field

## ➤ ISO 8529 Reference neutron radiation

Neutron radiations for determining the response of neutron-measuring devices  
as a function of neutron energy

$E_n$ (MeV)	Method of production
Thermal	Moderated (Reactor, accelerator, RI source) neutrons
0.002	Scandium-filtered reactor beam or $^{45}\text{Sc}(p,n)^{45}\text{Ti}$
0.024	Iron filtered reactor neutron beam or $^{45}\text{Sc}(p,n)^{45}\text{Ti}$
0.144	Silicon-filtered reactor neutron beam or $\text{T}(p,n)^3\text{He}$ , $^7\text{Li}(p,n)^7\text{Be}$
0.25	$\text{T}(p,n)^3\text{He}$ , $^7\text{Li}(p,n)^7\text{Be}$
0.565	$\text{T}(p,n)^3\text{He}$ , $^7\text{Li}(p,n)^7\text{Be}$
1.2	$\text{T}(p,n)^3\text{He}$
2.5	$\text{T}(p,n)^3\text{He}$
2.8	$\text{D}(d,n)^3\text{He}$
5.0	$\text{D}(d,n)^3\text{He}$
14.8	$\text{T}(d,n)^4\text{He}$
19.0	$\text{T}(d,n)^4\text{He}$

# 중성자공간선량당량계 (IEC61005)

## Reference neutron radiation for general test

- $^{252}\text{Cf}$ ,  $^{241}\text{Am-Be}$ ,  $\text{D(d,n)}^3\text{He}$
- Thermal or epi-thermal neutrons
- Workplace neutron fields

## For variation of response

- thermal
- 1 neutron energy for 1keV ~50keV
- 1 neutron energy for 50 keV ~ 600 keV
- 1 neutron energy for 1 MeV ~ 5 MeV
- 1 neutron energy for 13.5 MeV ~ 16 MeV

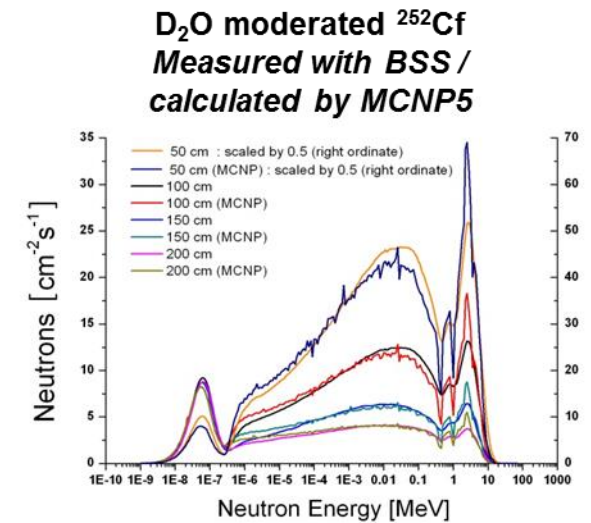
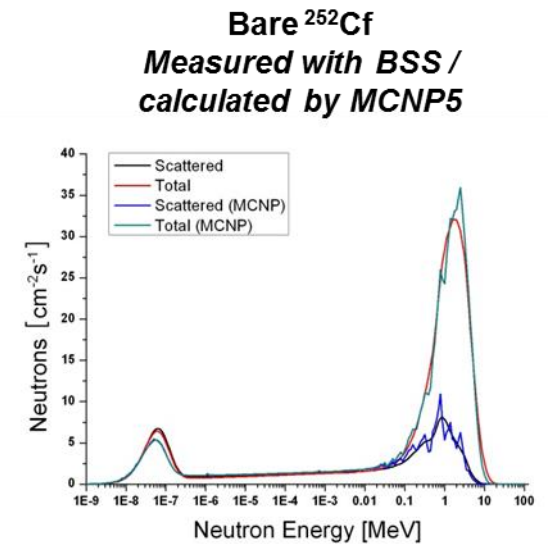
# 개인선량계 시험 (IEC61526)

Neutron response : 0.025eV ~ 15 MeV

시험용 중성자선원

- thermal
- 1 mono-energy for 10keV~100keV
  - Workplace field로 대체 가능
- 3 mono-energy for 100 keV ~ 1 MeV
- 3 mono-energy for 1 MeV ~ 10 MeV  
or 2 mono-energy +  $^{252}\text{Cf}$ (or  $^{241}\text{Am-Be}$ )
  - 1개 에너지 work place field or broad source로 대체 가능
- 1 mono-energy for 10 ~ 15 MeV
- If response range is above 15 MeV, 1 shall be used

# Standard Neutron field at KRISS

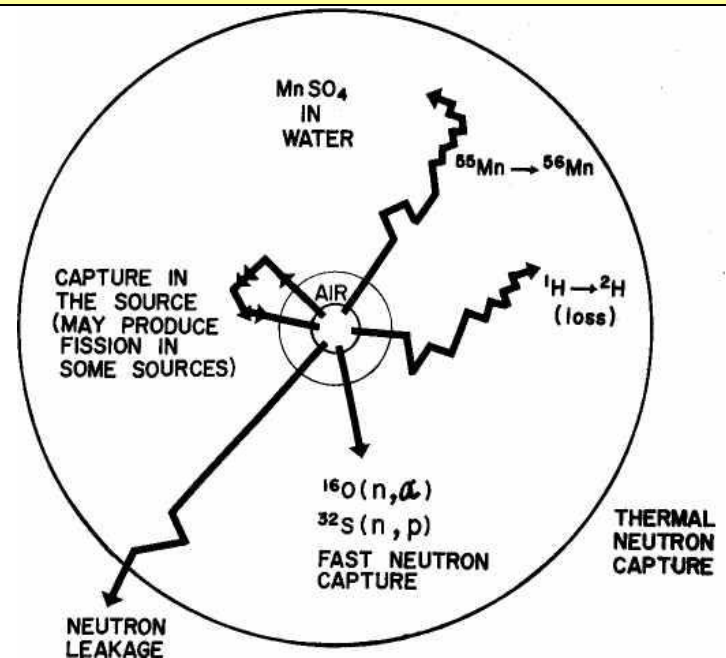




# Neutron emission rate 측정

Manganese Sulphate Bath system

For absolute measurement of Neutron emission rate



$$B = \frac{1}{f} \times R$$

$B$  : neutron emission rate

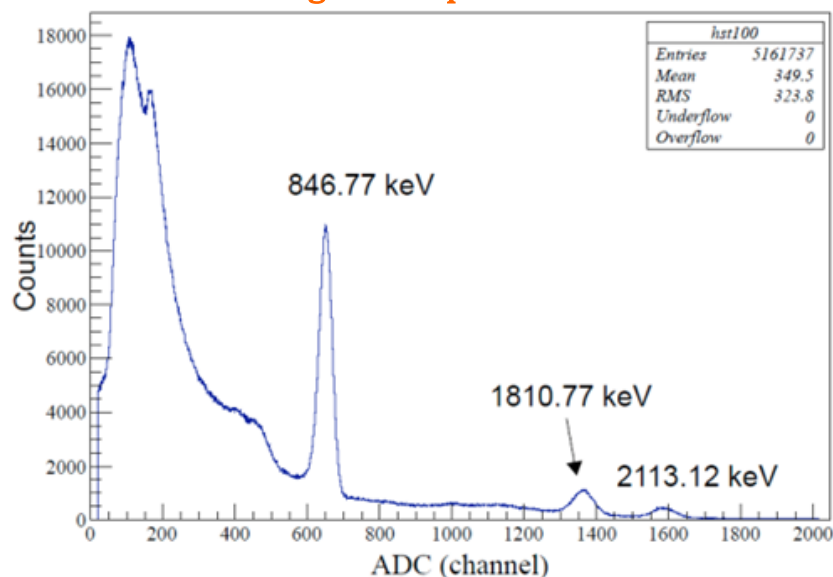
$R$  : production rate of  $^{56}\text{Mn}$

← can be calculated from the saturated activity of  $^{56}\text{Mn}$

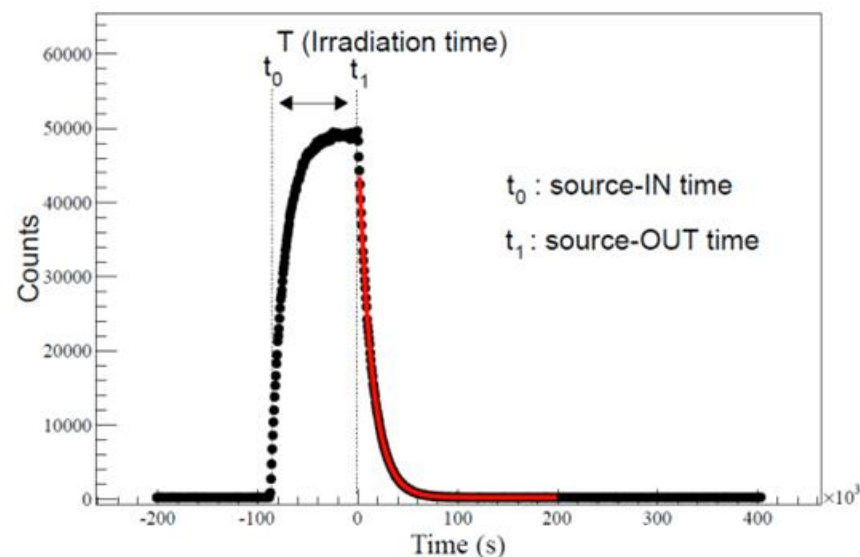
$f$  : neutron capture probability by  $^{55}\text{Mn}$

# $\gamma$ from $^{56}\text{Mn}$ decay

$^{56}\text{Mn}$  gamma spectrum



Gamma count rates



$$B = \frac{1}{f} \times R$$

$B$  : neutron emission rate

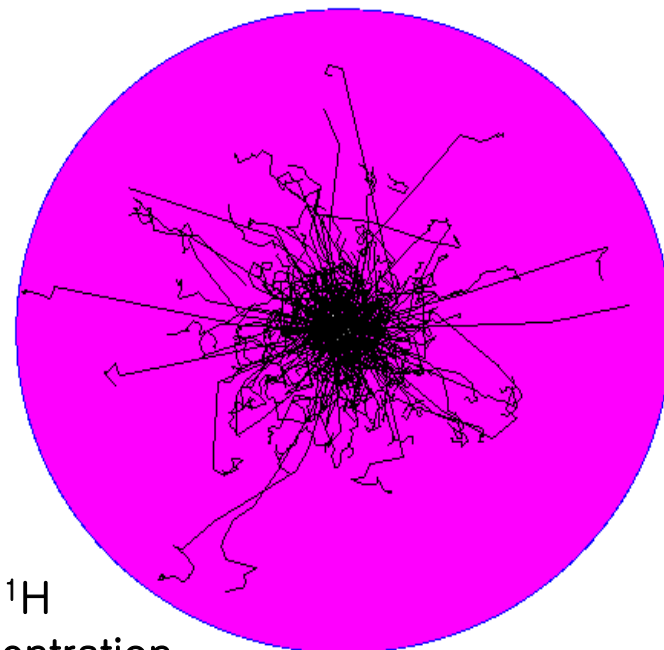
$R$  : production rate of  $^{56}\text{Mn}$

← can be calculated from the saturated activity of  $^{56}\text{Mn}$

$f$  : neutron capture probability by  $^{55}\text{Mn}$

# Neutron capture probability by $^{55}\text{Mn}$

	Prob. Which ONE neutron captured by $^{55}\text{Mn}$
Cf252-I	0.10508
Cf252-II	0.10485
AmBe (5Ci)	0.10186
AmBe (100mCi)	0.10261



Uncertainty : 0.5% from cross section ratio of  $^{55}\text{Mn}$  and  $^1\text{H}$   
 0.15% from the uncertainty of the  $^{55}\text{Mn}$  concentration

$$B = \frac{1}{f} \times R$$

$B$  : neutron emission rate

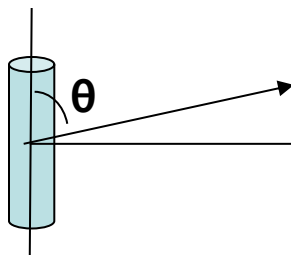
$R$  : production rate of  $^{56}\text{Mn}$  ← from the saturated activity of  $^{56}\text{Mn}$

$f$  : neutron capture probability by  $^{55}\text{Mn}$

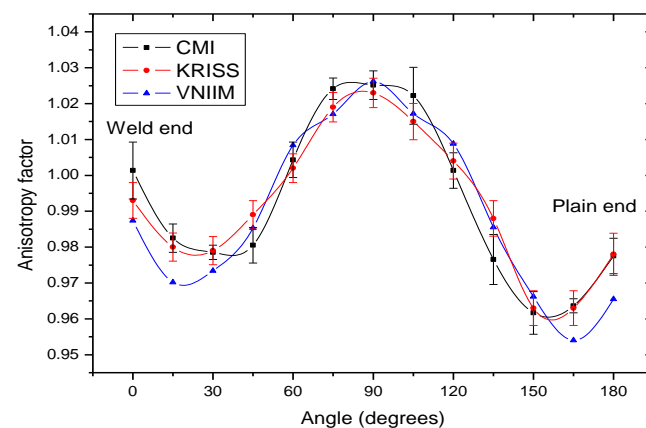
← estimated by MCNP calculation

# Neutron emission rate 국제비교

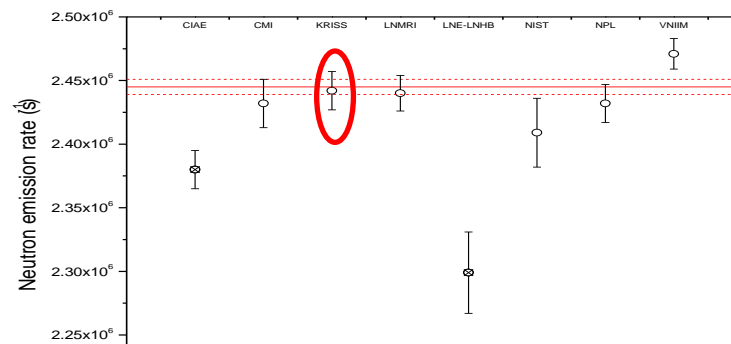
Manganese Sulphate Bath system  
For absolute measurement of Neutron emission rate



Anisotropy for  $^{241}\text{Am}/^9\text{Be}$  : CCRI(III)-K.9

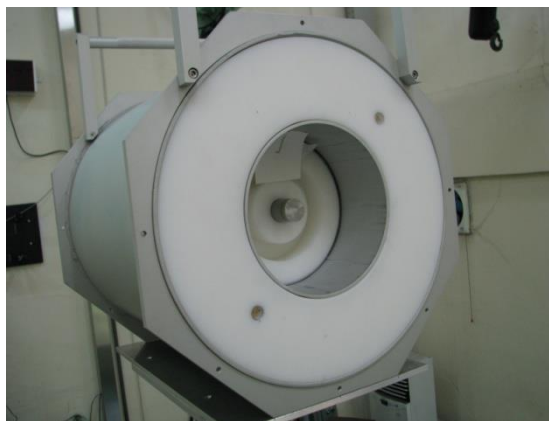


Emission Rate for  $^{241}\text{Am}/^9\text{Be}$  : CCRI(III)-K.9

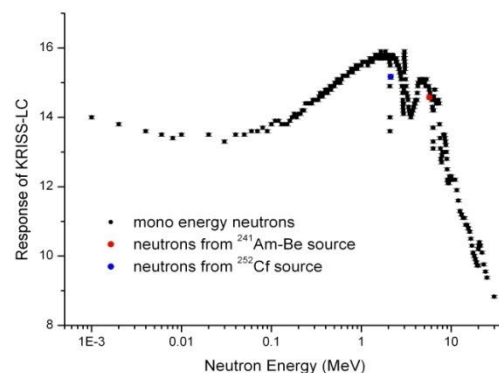


# Fluence rate measurement : Long counter

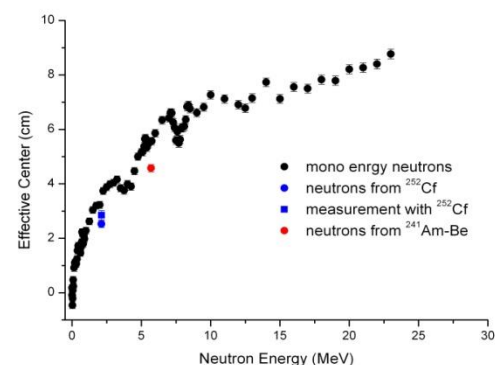
## Long Counter for the neutron fluence measurement



Response function of KRIS-LC



KRIS-LC Effective center



Neutron fluence :  $\phi = M/R$  at  $r$  (distance from the source to the effect center of LC)

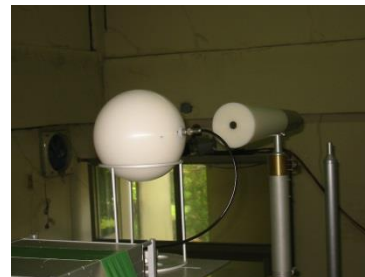
$M$  : Count rate measured by Long counter (1/s)

$R$  : Response of Long counter ( $\text{cm}^2$ )

Practically, the count rate should be like

$$M = M_{total} - M_{scattered}$$

$M_{scattered}$



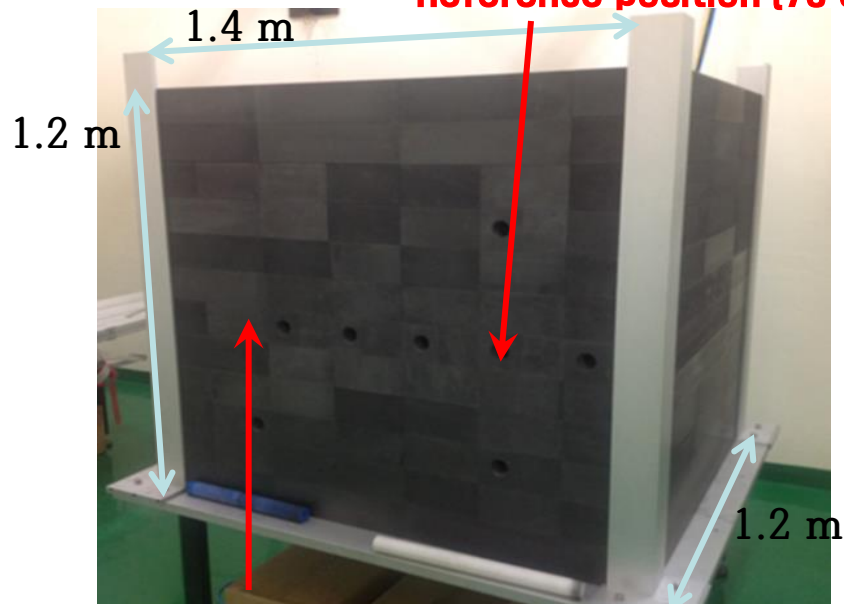
$M_{total}$





# Thermal neutron field at KRISS

Reference position (70 cm from source)



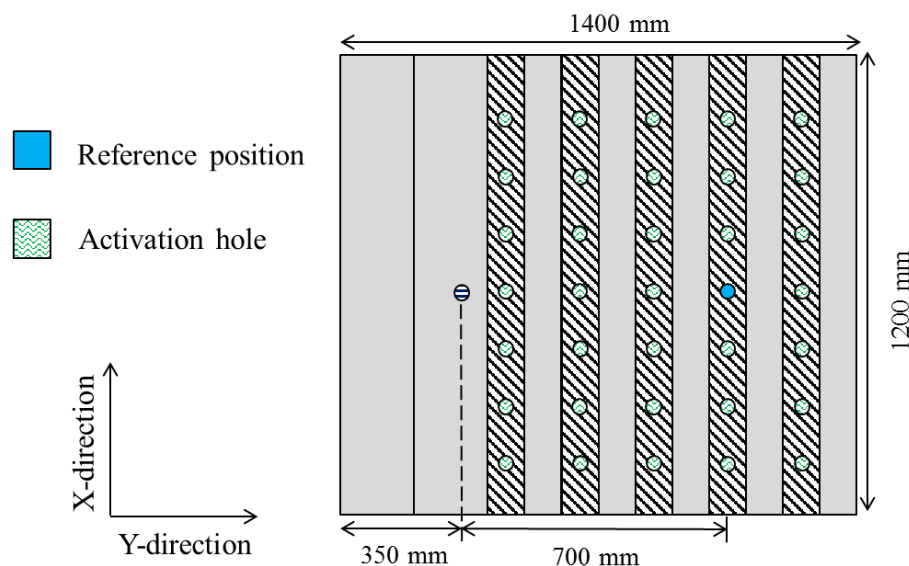
Source position (35 cm from the center)

## 고순도 그래파이트 파일 제작

- 1.4 m x 1.2 m x 1.2 m (239개 블록)
- 보론 불순물 함유량  $0.628 \pm 0.028 \mu\text{g/g}$   
PGAA 방법으로 분석
- 기타 불순물 검사 : GDMS 법으로 검사
- 밀도 측정 : 139개 블록 질량/부피 측정  
 $1.786 \pm 0.015 \text{ g/cm}^3$
- 중성자 선원 :  $^{241}\text{Am}-\text{Be}(\alpha, n)$  선원



# Thermal neutron field at KRISS



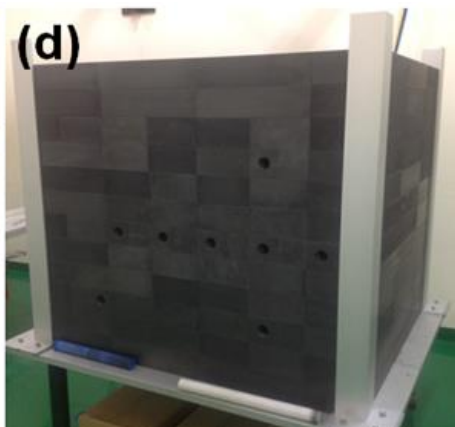
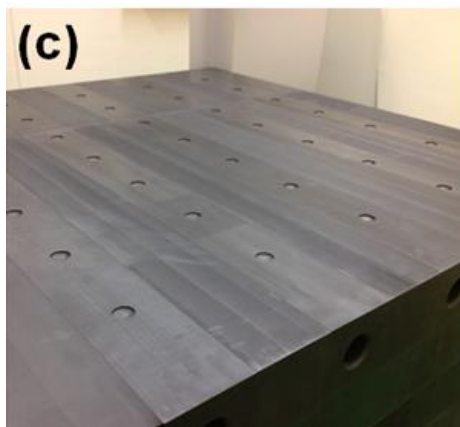
To specify neutron field

1. Monte-Carlo Simulation with MCNP  
: neutron energy distribution  
and various correction factors

2. Measurement  
with Gold foil activation method  
: neutron fluence rate



$$D = N\sigma_0 nv_0 \rightarrow \phi_w = nv_0 = \frac{D}{N\sigma_0}$$



# Neutron energy distribution(MCNP)

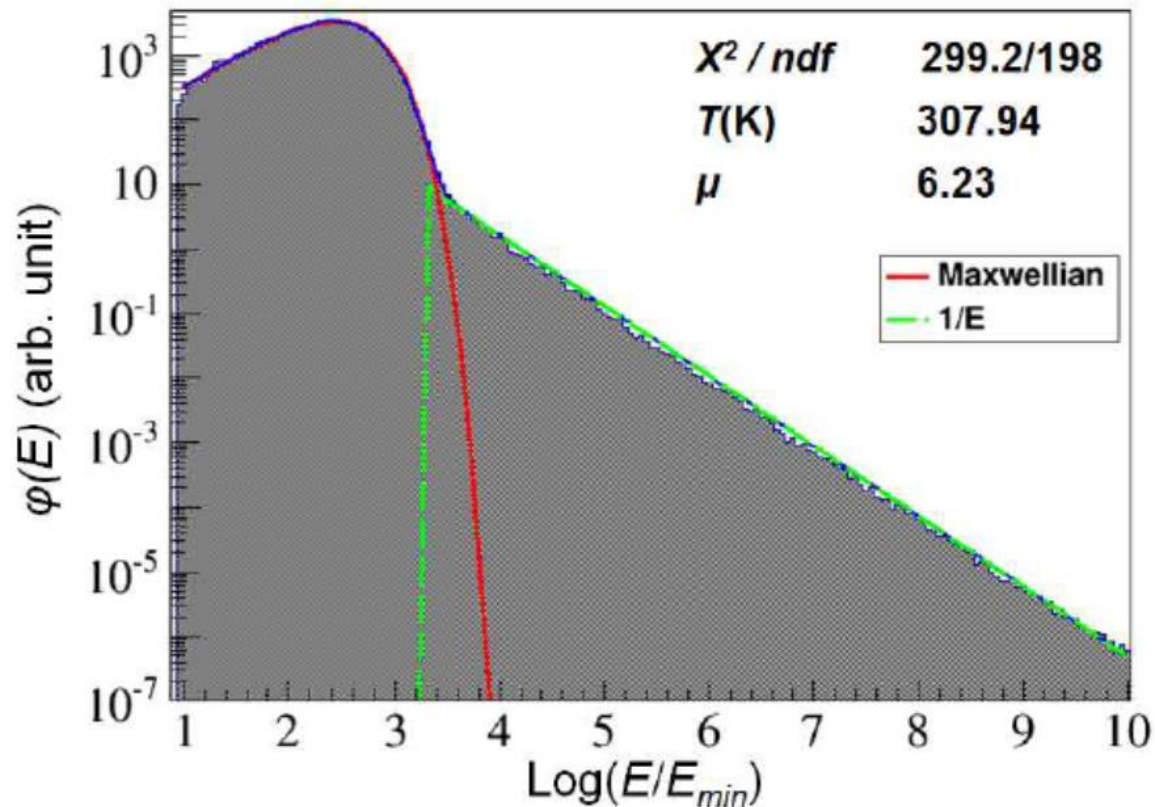
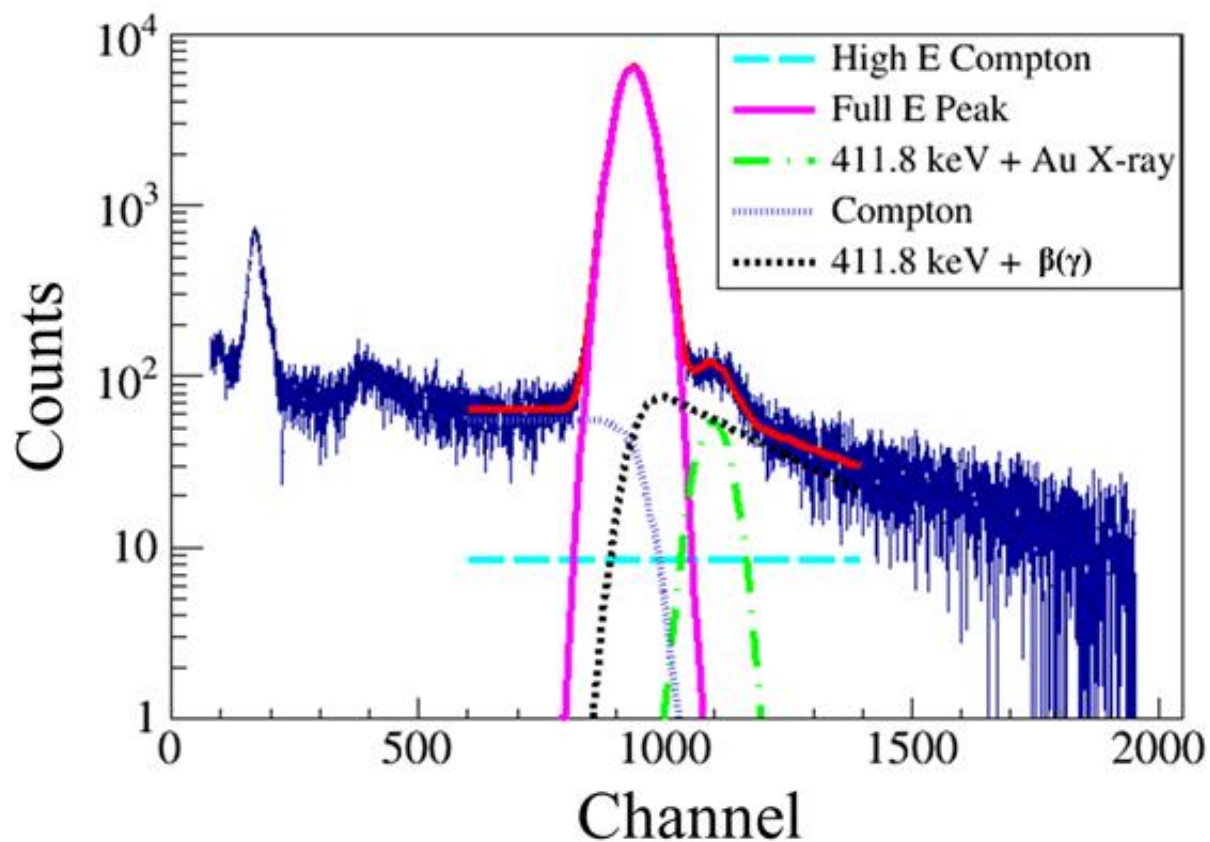


Figure 4.2. Fitting the spectrum obtained from MCNPX ( $E_{min} = 10^{-4}$  eV)



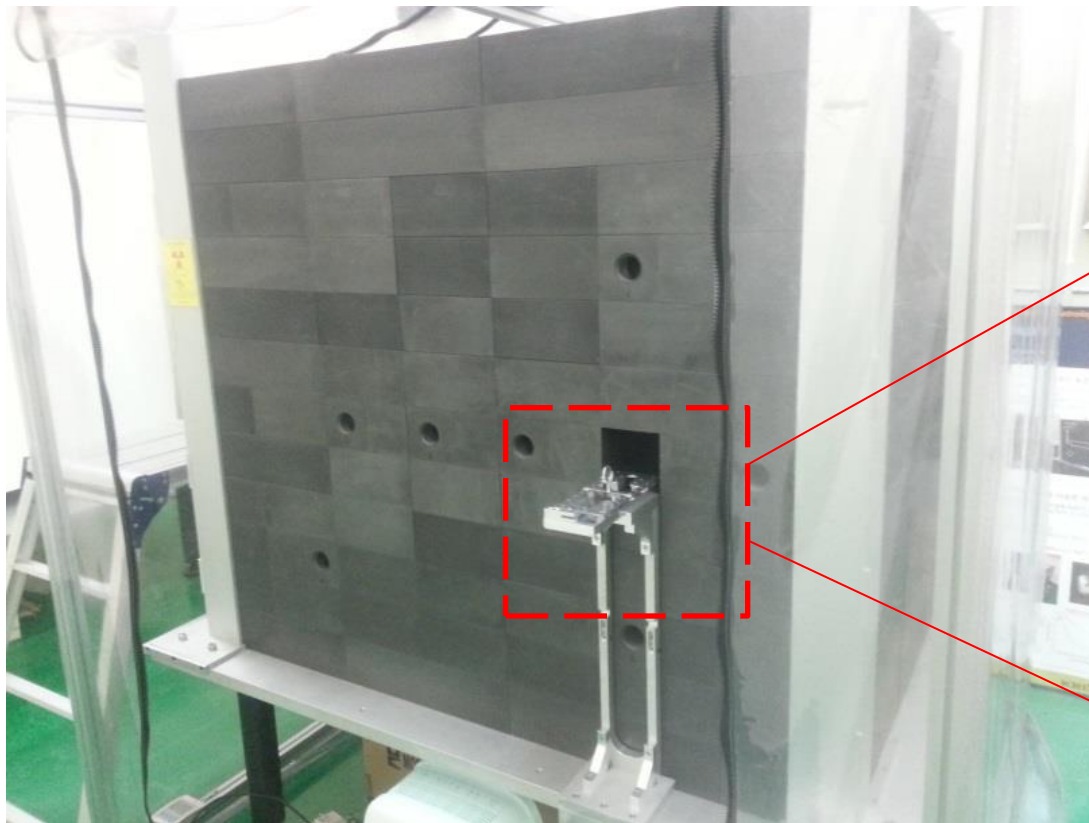
# Gamma energy spectrum from $^{198}\text{Au}$ decay



# Thermal neutron fluence

	Unit	Value	Uncertainty (%)
$D_0$	Bq/mg	0.6749	0.28
$D_0(\text{Cd})$	Bq/mg	0.0161	1.06
$T$	K	308	2.00
$\mu$		6.3	50.0
$\varphi_M$	$\text{cm}^{-2}\text{s}^{-1}$	2683.8	
$\varphi_{1/E}$	$\text{cm}^{-2}\text{s}^{-1}$	14.5	
$\varphi_{th}$	$\text{cm}^{-2}\text{s}^{-1}$	2698.3	1.5

# Thermal neutron field at KIRSS

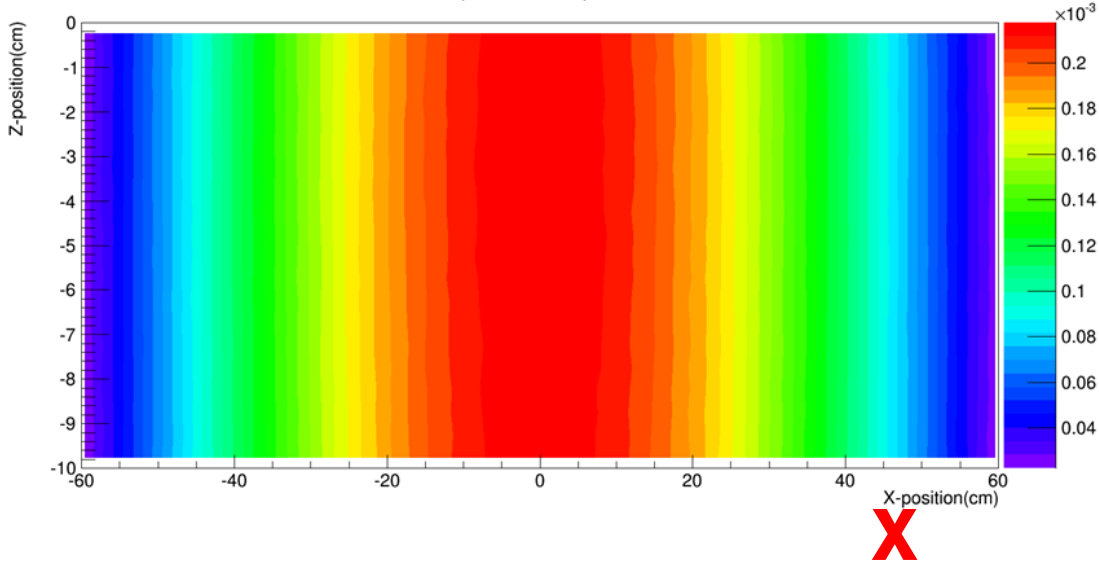


# Thermal neutron fluence inside cavity

	Unit	Value	Uncertainty (%)
$D_0$	Bq/mg	0.6040	0.33
$D_0(\text{Cd})$	Bq/mg	0.0186	2.01
$T$	K	311	2.00
$\mu$		6.3	50.0
$\varphi_M$	$\text{cm}^{-2}\text{s}^{-1}$	2354.4	
$\varphi_{1/E}$	$\text{cm}^{-2}\text{s}^{-1}$	16.8	
$\varphi_{th}$	$\text{cm}^{-2}\text{s}^{-1}$	2371.2	1.5

# Thermal neutron field inside cavity

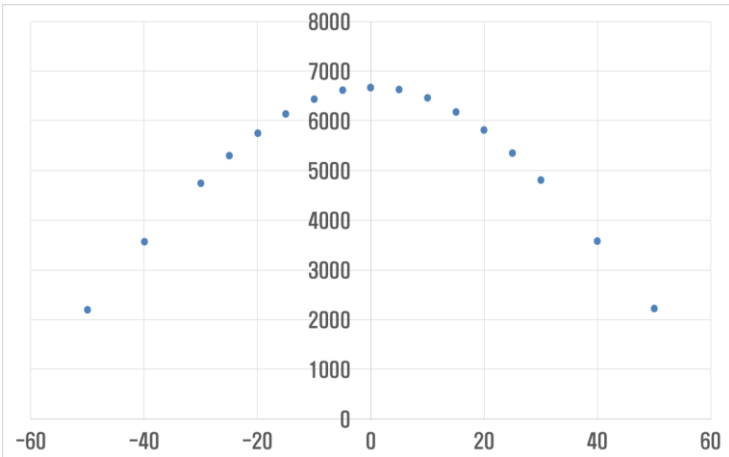
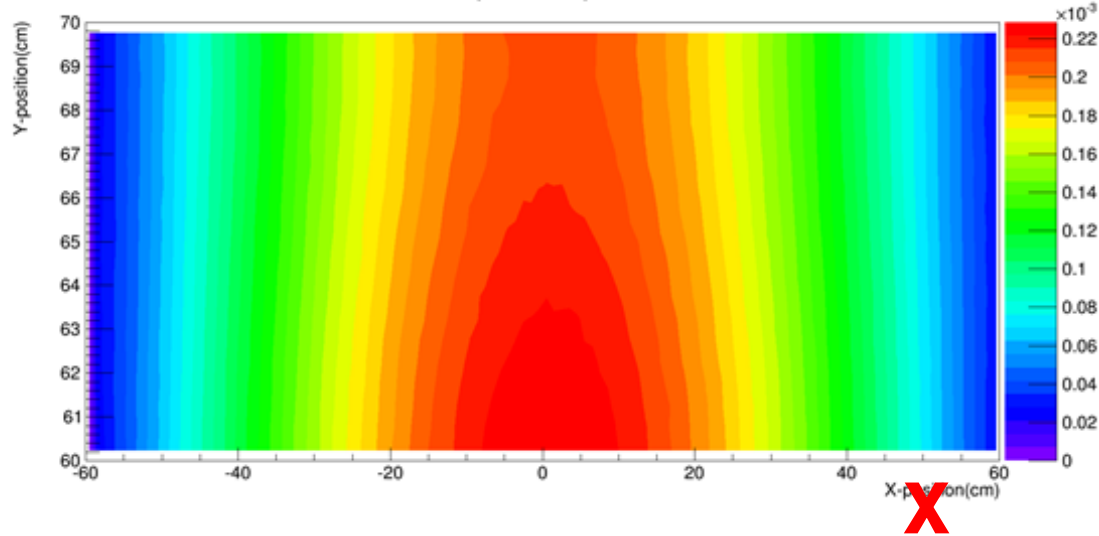
Neutron fluence rate ( $E < 0.6\text{eV}$ ) in P4 line  $Y = 65.25\text{ cm}$



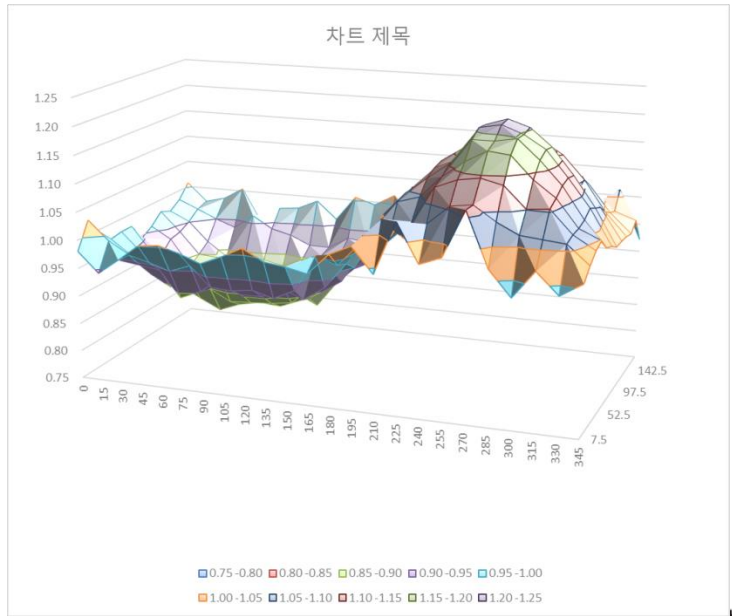
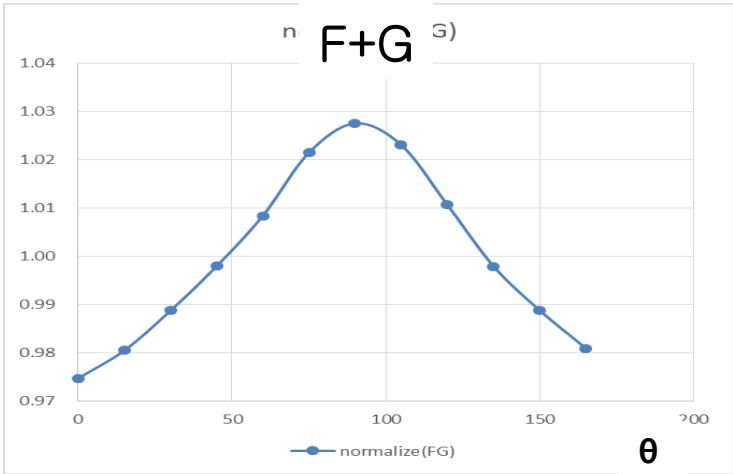
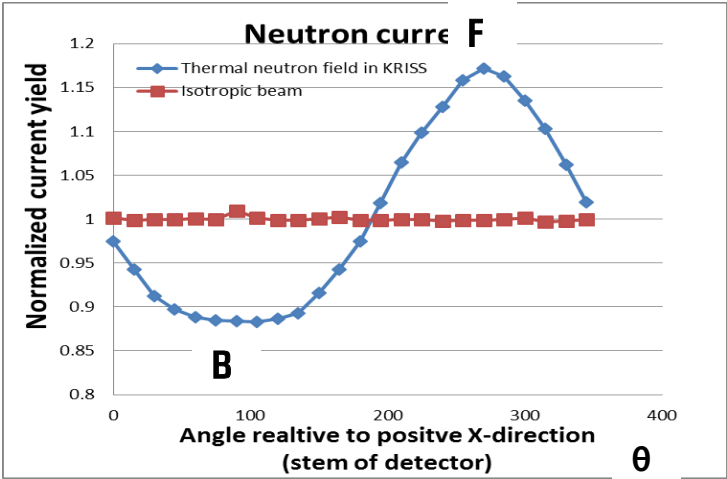
## Uniformity check



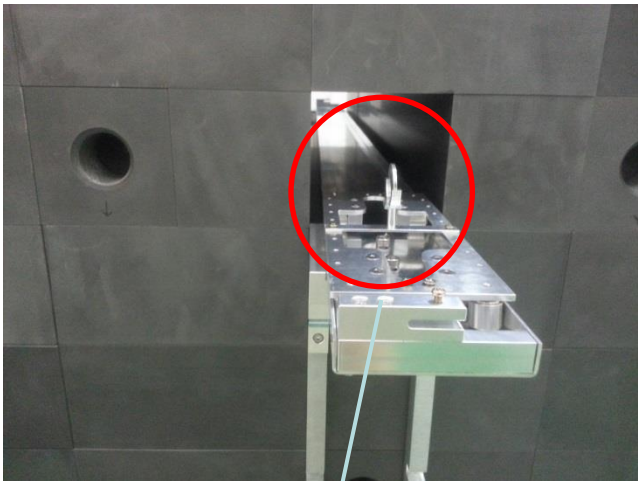
Neutron fluence rate ( $E < 0.6\text{eV}$ ) in P4 line  $z = -4.75\text{ cm}$



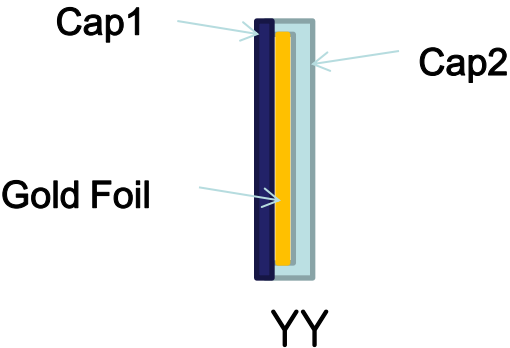
# Isotropy of thermal neutron inside



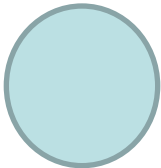
# Isotropy of thermal neutron inside cavity



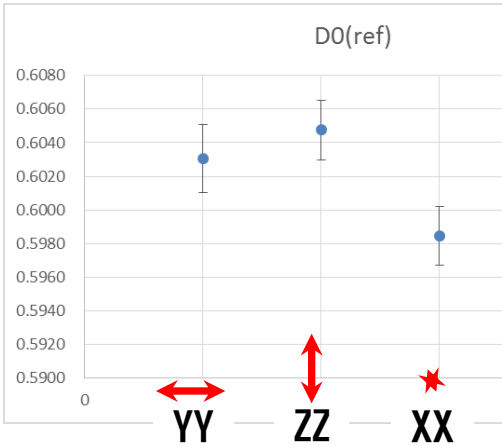
Gold foil (20 $\mu$ m thick, radius 1cm)  
in case (Case1+Case2)  
made of Al and/or Cd  
Cap1 : side of <sup>241</sup>Am-Be source  
Cap2 : opposite side



ZZ



XX



Cap1	Cap2	Specific activity <sup>198</sup> Au	Relative to the ref case (A)
Al	Al	0.6031 (0.3%)	1
Al	Cd	0.3245 (0.4%)	53.8%
Cd	Al	0.2540 (0.4%)	42.1%



# SP9 sensitivity

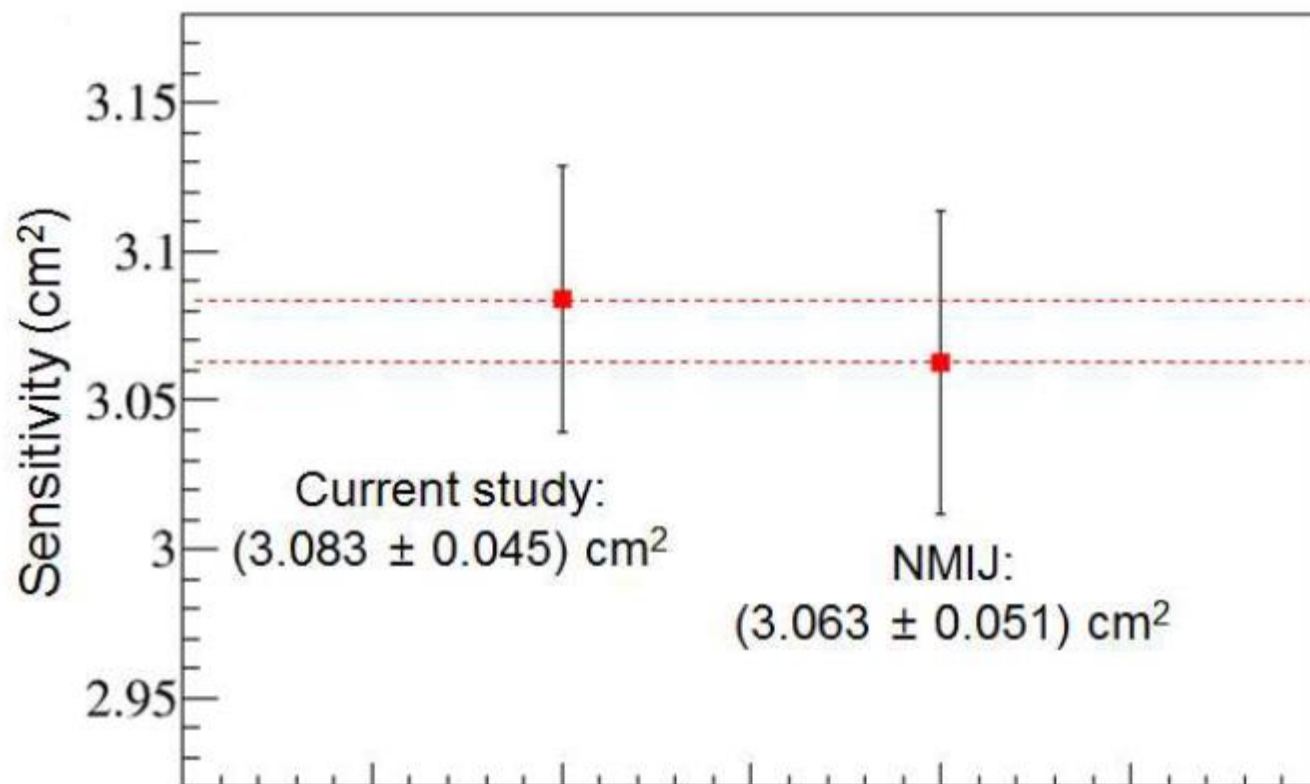
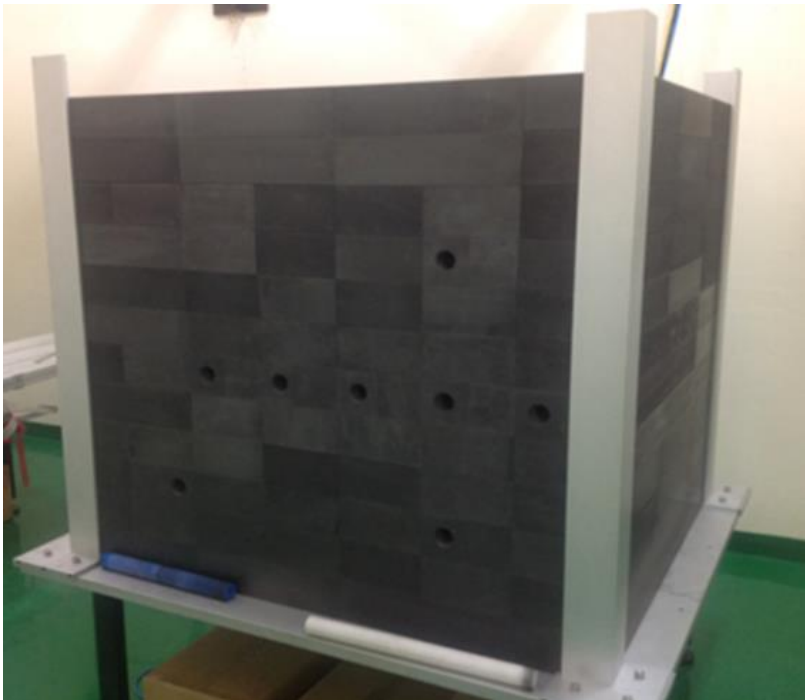


Figure 5.6. Determined sensitivity using the developed thermal neutron

field and the certified sensitivity from NMIJ;  $R_{ref} = R_{th} \times \prod_{i=1}^3 k_i$



# outside graphite pile



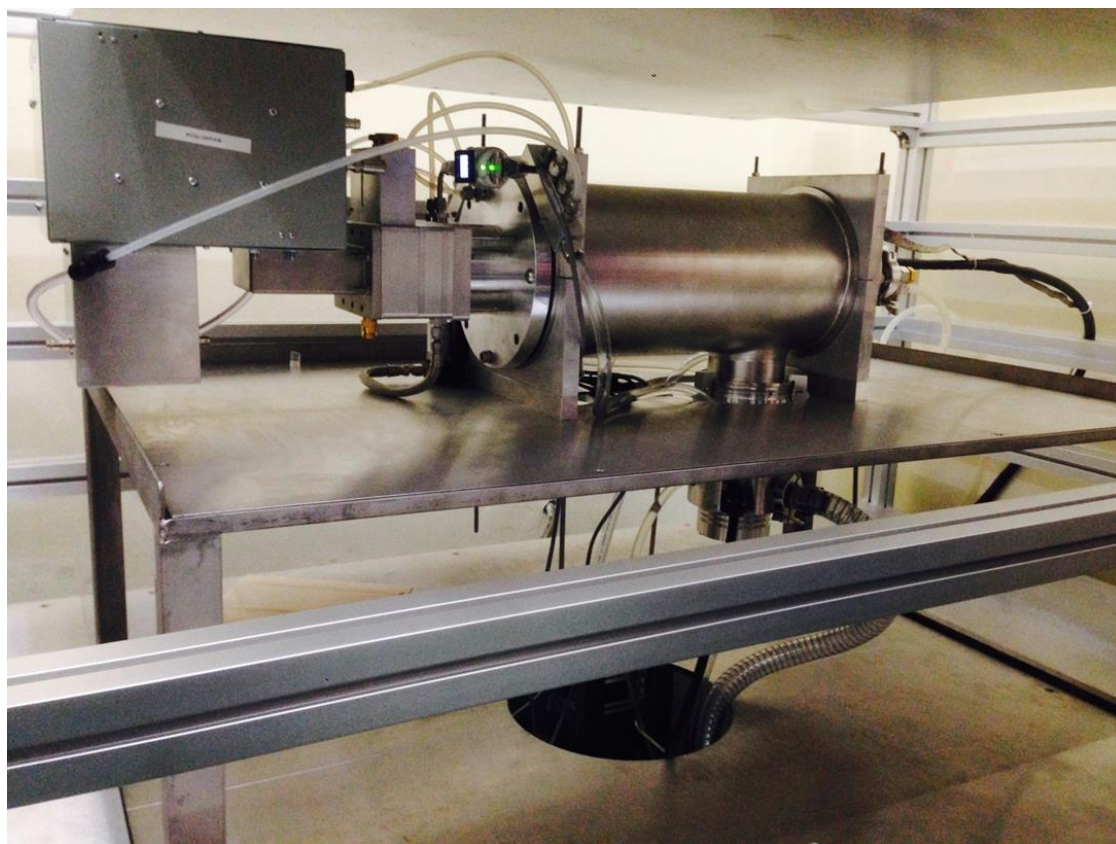
	Unit	Value	Uncertainty (%)
Counts	1/s	185.4	0.62
Counts(Cd)	1/s	0.7	1.6
$R_{SP9}$	cm <sup>2</sup>	2.810	1.6
에너지 차이(k1)		1.0	0.5
입사각차이(k2)		1.034	0.33
공동/오픈(k3)		1.028	0.68
$\phi_{eff}$	cm <sup>2</sup> s <sup>-1</sup>	61.8	2.0



# Neutron generator at KRISS

## Neutron generator

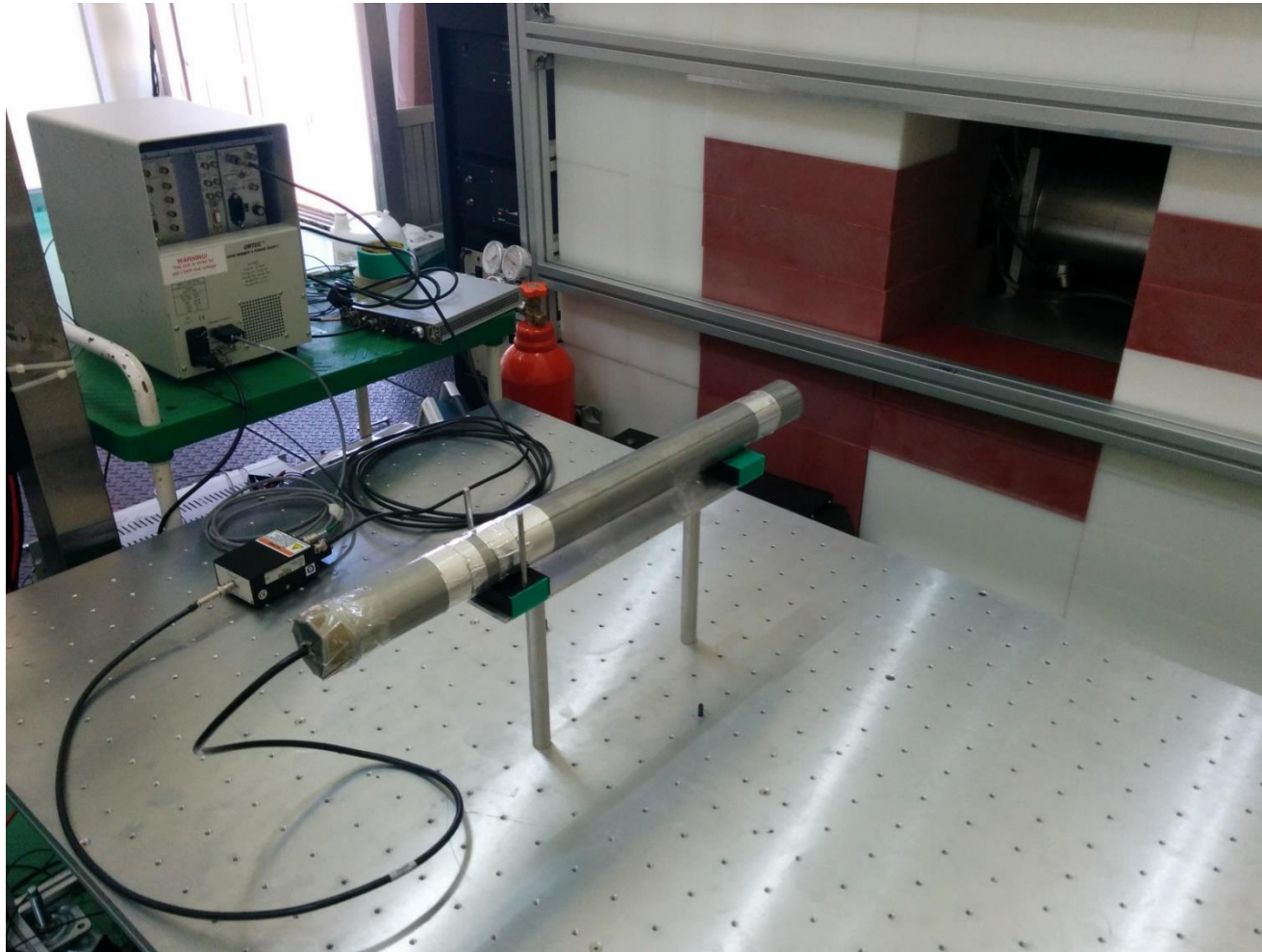
- Produced by Adelphi (DD-109 model) :
  - $d(d,n)t$  reaction, ( $E_d = 85\sim 125$  keV)



# Neutron generator at KRISS

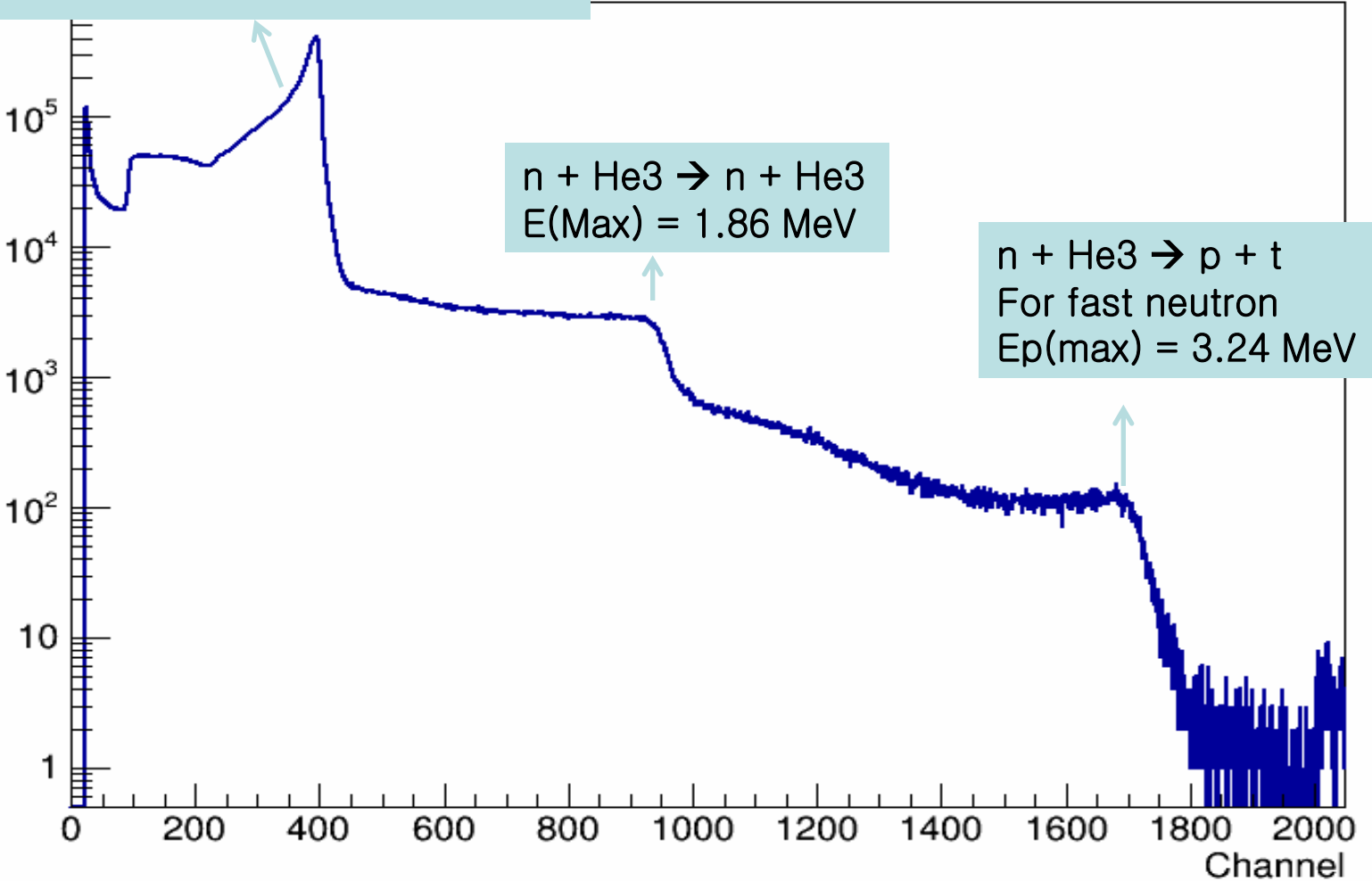
$^3\text{He}$  proportional counter for neutron energy measurement

→  $E_n = 2.46 \text{ MeV}$

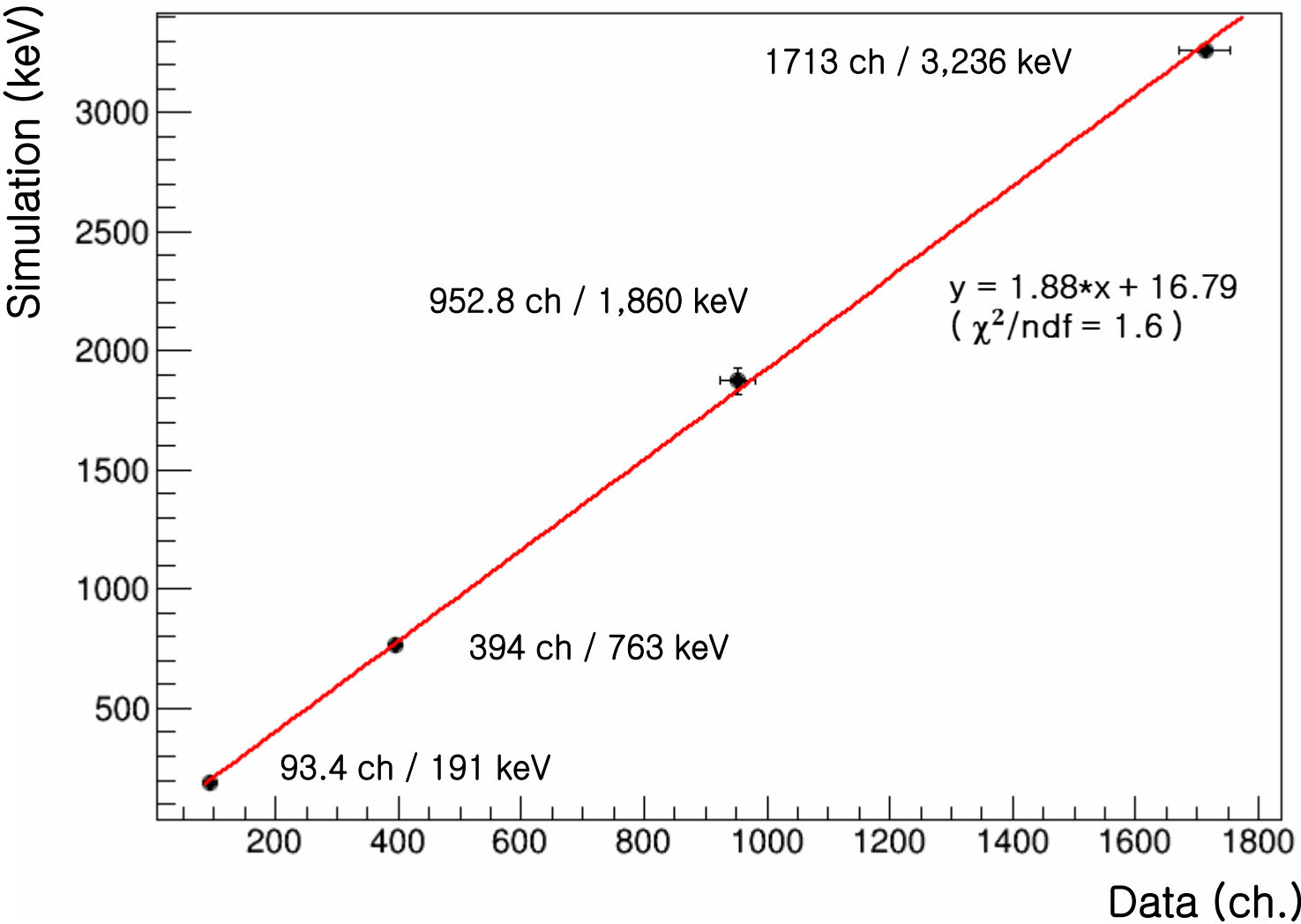


# He3 detector spectrum

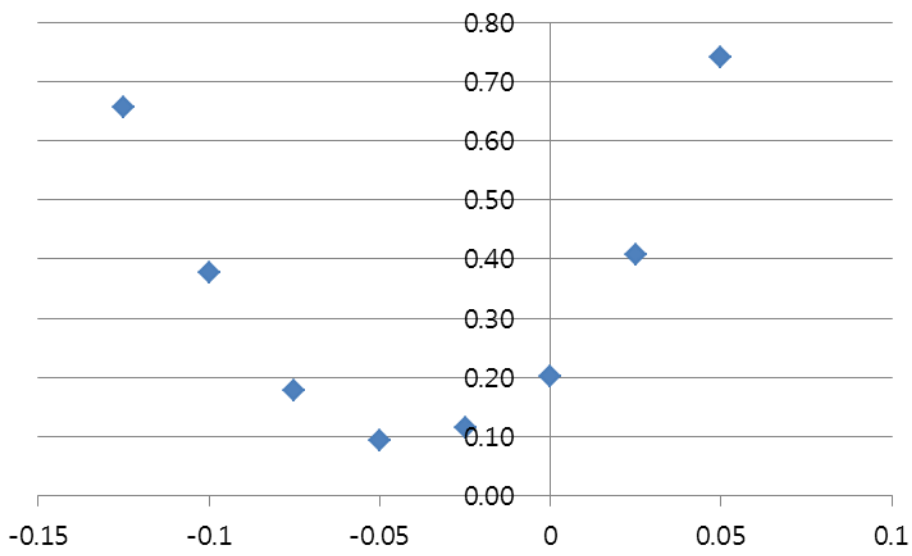
$n + \text{He}3 \rightarrow p + t$   $Q = 764 \text{ keV}$   
For thermal neutron







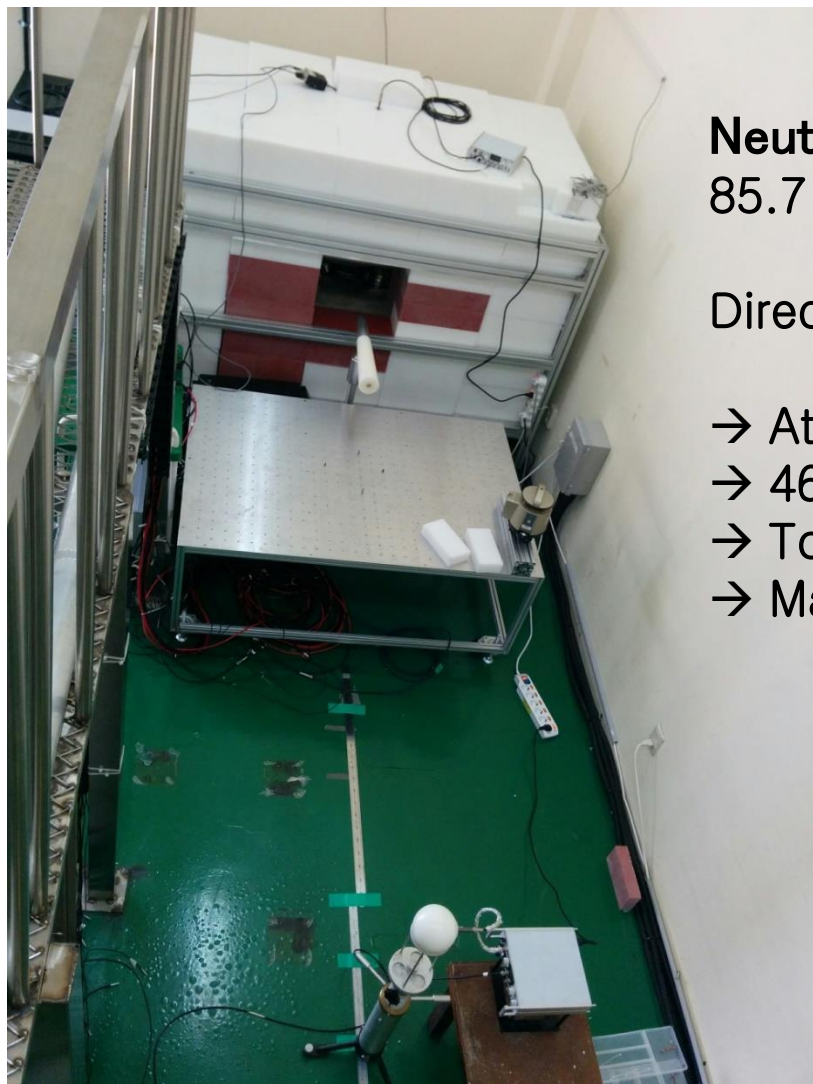
Energy difference (MeV)	Proton escape		Thermal n full energy peak		3He recoil		p+t full energy peak		Const.	Slope	$\chi^2/\text{ndf}$
	Data	Sim	Data	Sim	Data	Sim	Data	Sim			
- 0.1	93.4	191	393	763	953	1785	1713	3136	17.46	1.86	<b>0.38</b>
- 0.05						1823		3186	14.54	1.89	<b>0.09</b>
$\pm 0$						1860		3236	11.67	1.92	<b>0.20</b>
+ 0.05						1898		3286	8.78	1.94	<b>0.74</b>
+ 0.1						1936		3333	5.99	1.97	<b>1.57</b>



Neutron Beam energy  
= 2.43 ± \*\*\* (MeV)

X axis : Neutron energy difference  
Y axis : chi-sqr / ndf

# Neutron generator at KRISS



## Neutron fluence measurement with BS

$85.7 \pm 2.7$  (3 %) /cm<sup>2</sup>/s at 3.5 m from the target

Direct vs scatter = 1 : 1 at 3.5 m

→ At 1.5 m, Direct vs scatter = 5 : 1 (expected)

→ 460 /cm<sup>2</sup>/s

→ Total emission rate =  $\sim 1.5 \times 10^8$  /s

→ Max power : we can have 5 times more

# Plan for new laboratory

**2017 ??**

Low Scatter Neutron Irradiation Room  
12 m x 12 m x 12 m

**2018 ??**

Cockcroft–Walton  
Accelerator (500 kV)

**2019-2020??**

2.5 MeV & 14.8 MeV 단색중성자 표준장 확보