Development of Cyclic Pressurization Fatigue Test Technique for Spent Fuel Cladding Tube

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1. Introduction

Nuclear fuel cladding undergoes a cyclic deformation due to various kinds of external parameters during inreactor operation. Nowadays, the potential occurrences of a high cycle fatigue are considerably decreased by virtue of improvement of a spacer grid in a fuel design. However, it is well known that the possibility of a low cycle fatigue along the radial direction which is caused by power oscillations still remains in the fuel cladding.[1,2] If the utility adopts a load following operation, the cyclic changes of the diameter causing a low-cycle fatigue will occur more frequently. Although failures regarding a radial fatigue in the fuel cladding have not been reported yet, it is essential to accumulate a fatigue life database for use in a fuel design. Since Soniak's proposal for the low cycle radial fatigue under cyclic pressurization of the fuel cladding, KAERI's R&D group has also produced a lot of low cycle fatigue data for the un-irradiated fuel cladding tube using a cyclic pressurization device.[3] However, fatigue data regarding irradiated fuel cladding under cyclic pressurization has not been obtained around the country until now. And the infrastructures and fatigue test techniques, which can produce the fatigue data on the irradiated fuel cladding, are still worse off.

The objectives of this study are to develop a low cycle fatigue test techniques for irradiated fuel cladding, as well as to produce a stress-life curve of the irradiated cladding under the cyclic pressurization.

2. Methods and Experiment

2.1 Development of Cyclic Pressurization Fatigue Tester

The cyclic pressurization fatigue test machine consists of five major components, such as main frame, electric furnace, control system, hydraulic supply module, and data acquisition system. Figure 1 shows a schematic diagram of the cyclic pressurization fatigue tester for an irradiated fuel cladding tube. The hydraulic booster cylinder moves up and down to load a cyclic pressure to the irradiated cladding specimen. The pressure of the cladding can be controlled within the range of 0 to 126 MPa by the hydraulic servo valve, and the resultant hoop stress ranges up to 992 MPa. Silicon oil which is resistant to boiling, degradation, and reactivity under high temperature condition was used as a medium to exert an internal pressure into the irradiated cladding tube. The loading frequency can be controlled in a range of 0.5 to 2 Hz with a sawtooth and

sinusoidal waveform. The electric furnace heater was divided into two independent control zone to maintain the uniform temperature distribution axially along the irradiated cladding specimen.

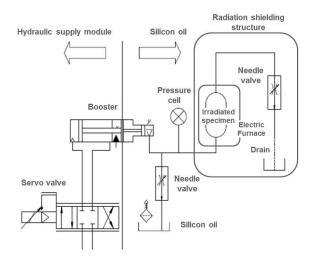


Figure 1. Schematic diagram of cyclic pressurization fatigue tester

Radiation shielding structures that surround the irradiated cladding specimen and the electric furnace were installed to protect an experimentor and to minimize the radiation exposure from the spent fuel cladding specimens. The history of spent fuel cladding for source term analysis is as follows;

- Initial enrichment of U-235: \leq 5 wt%
- Rod average discharge burn-up: 60 GWd/tU
- Cooling period: ≥ 2 years
- Cladding material: Zirlo
- Defueled cladding with 250mm length

Based on the shielding calculation results which are produced by the MCNP5 code system, the shielding material is determined to the rectangular shaped pure lead with 50mm thickness. And the lead structure is covered with 5mm thick stainless steel casing.

2.2 Performance Test of Fatigue Tester

Performance test of the newly developed cyclic pressurization fatigue tester was carried out on the unirradiated cladding tube specimens. A constant pressure difference was applied to the un-irradiated zircaloy fuel cladding tube. A sawtooth waveform was applied, where the maximum internal pressure was varied from 44.8 MPa to 64.1 MPa, while the minimum internal pressure was held constant at 10 MPa. The durability of the tester was good up to 10^6 cycles with 1 Hz frequency.

To ensure the uniform temperature distribution along the specimen, seven thermocouples were attached on the surface of cladding specimen at every 1 inch. As shown in figure 2, the temperature gradients in 100mm distance around the middle of specimen are with ± 2 °C.

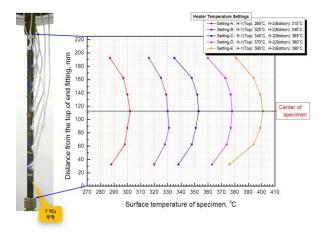


Figure 2. Axial temperature distribution along cladding specimen

2.3 Low cycle fatigue test under cyclic pressurization

Using the developed test machine, fatigue behavior data of the un-irradiated advanced zircaloy cladding tube w/ and w/o hydrogen have been produced from this study. And the preliminary fatigue test for irradiated advanced zircaloy cladding tube was also carried out.

A low-cycle fatigue test, where a constant pressure difference was applied to the un-irradiated advanced fuel cladding (PLUS7, 17ACE7) tube, was performed. A sawtooth waveform was applied, where the maximum hoop stress was varied from 350 MPa to 500 MPa, while the minimum hoop stress was held constant at 78 MPa. The temperature was maintained constant at 400° C during the fatigue test. The failure cycle, which corresponds to the maximum hoop stress, was measured to construct the S-N curve of the each cladding tube. The preliminary low-cycle fatigue test for the irradiated PLUS7 cladding tube was carried out in this study. The rod average discharge burn-up of the PLUS7 spent fuel is about 57 GWd/tU. And the cooling period is about 6 years. The low cycle fatigue test for irradiated PLUS7 cladding specimen was performed under the sawtooth waveform cyclic stress, where the maximum hoop stress was 400 MPa and the minimum hoop stress was held constant at 78 MPa, with 1 Hz frequency and 400 $^{\circ}$ C air environments.

3. Results

Figure 3 shows the stress-life diagram of the unirradiated and irradiated Advanced Zircaloy cladding tube under cyclic pressurization. An open mark means that a cladding tube specimen ruptured at the given cycle. A closed mark with an arrow means that a cladding tube specimen survived after the given cycles. As shown in figure, the failure cycle increased with decreasing applied stress. And it is found that the failure cycle of hydrided un-irradiated cladding tube specimen decreased drastically in the same level of maximum hoop stress in comparison with non-hydrided samples.

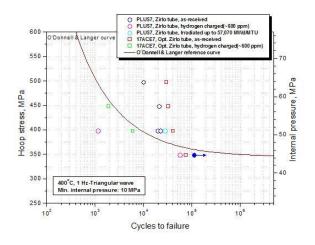


Figure 3. Stress-life curve of advanced Zircaloy cladding

REFERENCES

[1] K. Pettersson, "Low-cycle fatigue properties of Zircaloy-2 cladding," J. Nucl. Mater., 56, 91–102 (1975).

[2] P. R. Pandarinathan, P. Vasudevan, "Low-cycle fatigue studies on nuclear reactor Zircaloy-2 fuel tubes at room temperature, 300 and $350 \Box C$," J. Nucl. Mater., 91, 47–58 (1980).

[3] Jun Hwan KIM, et al., "Deformation Behavior of Zircaloy-4 Cladding under Cyclic Pressurization", Journal of Nuclear Science and Technology, 44:10, 1275-1280(2007)

[4] B. Adamson, Peter Rudling, "Mechanical Properties of Zirconium Alloys", IZNA-1 special topic report, 2002

[5] Soniak A., Lansiart S., Royer J., Mardon J. P. and Waeckel N. "Irradiation Effect on Fatigue Behavior of Zircaloy-4 Cladding Tubes", Zirconium in the Nuclear Industry, Tenth Int'l Symposium, ASTM STP 1245. A. M. Garde and E. R. Bradley, Eds., Am. Soc. for Testing and Materials, Philadelphia, 549-558, 1994.