

Development of a Module for the Evaluation of the Resonance Region

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1. Introduction

A new module has been developed to be added to a nuclear reaction model code EMPIRE [1] to allow for an evaluation of neutron cross sections in the resonance region. It automates most of the evaluation procedures and can be executed within EMPIRE or as a stand-alone program. The module performs an analysis of the available resonances, provides statistical distributions, and computes cross sections in the resolved and unresolved resonance regions. The module also provides an ENDF-6 formatted file [2] for the resonance region and various plots allowing for a verification of the procedure. The formatted file can be integrated later into the final ENDF-6 file as generated by the EMPIRE code. In the present paper, use of the resonance module and some sample cases are presented.

2. Calculation Method and Procedure

The resonance module consists of a graphic user interface (GUI) and a number of codes and scripts that read resonance parameters and other physical constants, perform an analysis of the available resonances, provide statistical distributions and compute cross sections in the resolved and unresolved resonance regions which are then compared with the available evaluated cross sections and experimental data.

At a startup, the module reads all the necessary data for the target nucleus including individual, as well as average, resonance parameters from the Atlas of

before performing the calculations in the resolved and unresolved regions. Figure 1 shows an example screen for Ni-60.

The PTANAL [5] and WRIURR [5] codes which have been modified for the resonance module are used for the calculation of resonance parameters in the resolved and unresolved energy regions, respectively. The Porter-Thomas distribution [6] is presented graphically by reading the output of the PTANAL code so that an user can easily determine the average level spacings and strength functions for the s- and p-waves

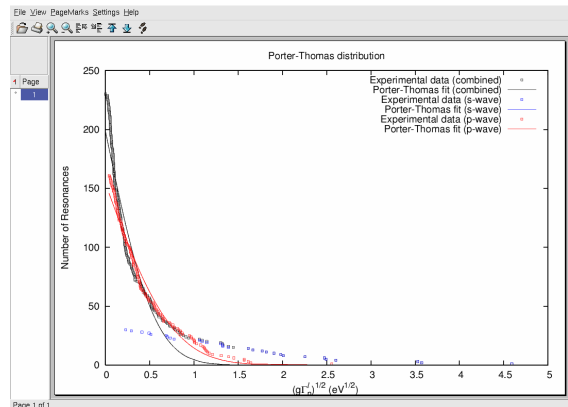


Figure 2. The Porter-Thomas distribution curves.

as shown in Figure 2. The SCANR code has been newly developed and is used for the graphical analysis of the resonance energies, which helps the determination of the upper boundary of the resolved resonance region

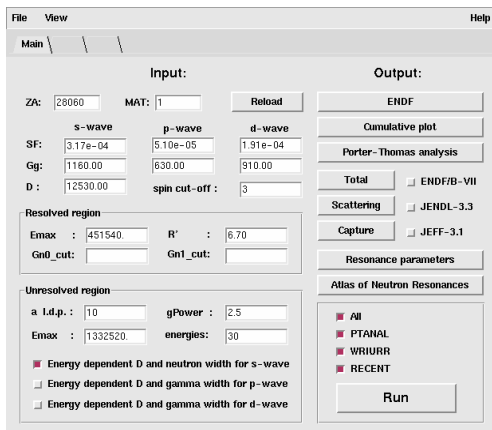


Figure 1. The screen shot of GUI main control panel.

Neutron Resonances [3] and other physical constants from RIPL-2 [4] and displays them on the screen. The user has the possibility to modify the nuclear parameters

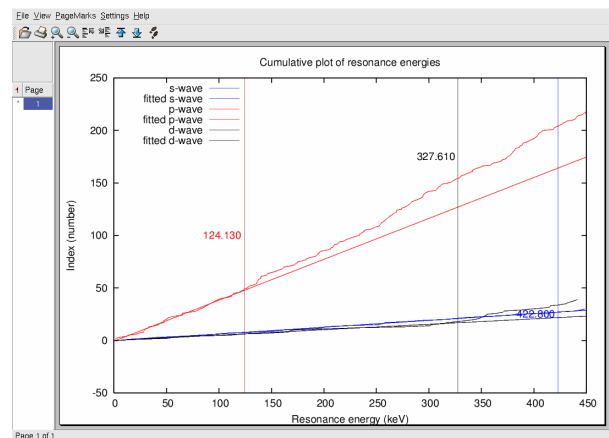


Figure 3. The cumulative plot of resonance energies.

(see Figure 3). The calculated resonance parameters can then be constructed as point-wise cross sections by calling the auxiliary codes such as RECENT [7] and

SIGMA1 [8] and compared with the experimental data and the available evaluated cross sections including ENDF/B-VII [9], JENDL-3.3 [10] and JEFF-3.1 [11] (see Figure 4).

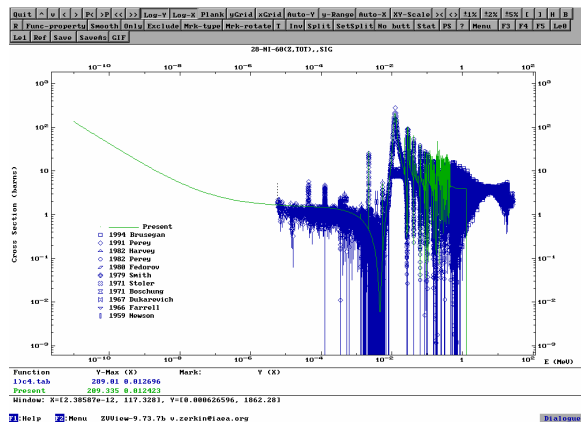


Figure 4. The screen shot of cross section comparison page.

All the jobs except the modification of default values for the standard resonance parameters and physical constants are executed with simple mouse clicks. The module also provides an ENDF-6 formatted file for the resonance region. The formatted file can be integrated later into the final ENDF-6 file as generated by the EMPIRE code.

3. Summary and Further Work

A new module has been developed for the evaluation of cross sections in the resolved and unresolved resonance regions. It automates most of the evaluation procedures and can be executed within EMPIRE or as a stand-alone program. When it is used as connected to the EMPIRE code, the single ENDF-6 formatted file can be obtained in the full energy region. As most of jobs are performed with a few simple mouse clicks on the GUI control panel providing various plots that help the user check the validity of the calculation results, it will minimize the possible human error factor and improve the productivity and efficiency of the evaluation tasks.

In the current version, the SCANR code searches and suggests the upper boundary of the resolved resonance region by comparing the chi squares between the experimentally determined resolved resonance energies and the fitted ones. While the sample evaluation works for some nuclides showed that the algorithm implemented in the SCANR code predicted the reasonable values, some other cases indicated the necessity for the improvement. The improvement to the algorithm will be made by inspecting the results in the future evaluation works for the more various nuclides.

REFERENCES

- [1] M. Herman, EMPIRE-II Statistical Model Code for Nuclear Reaction Calculations (Version 2.18 Mondovi), IAEA1169/06, 2002.
- [2] V. McLane, C. L. Dunford and P. F. Rose, ENDF-102: Data Formats and Procedures for the Evaluated Nuclear Data File ENDF-6, BNL-NCS-44945-01/04, 2001.
- [3] S. F. Mughabghab, Atlas of Neutron Resonances, Resonance Parameters and Thermal Cross Sections Z=1-100, 5th Edition, National Nuclear Data Center, Brookhaven National Laboratory, Upton, N.Y., 2006.
- [4] T. Belgia et al., Handbook for Calculations of Nuclear Reaction Data (Reference Input Parameter Library), IAEA-TECDOC-1034, 1998.
- [5] S. Y. Oh, J. H. Chang and S. F. Mughabghab, Neutron Cross Section Evaluation of Fission Products Below the Fast Energy Region, Brookhaven National Laboratory, BNL-NCS-67469, 2000.
- [6] C. E. Porter and R. G. Thomas, Fluctuations of Nuclear Reaction Widths, Physics Review, Vol. 104, p. 483, 1956.
- [7] D. E. Cullen, Program RECENT (version 79-1): reconstruction of energy dependent neutron cross sections from resonance parameters in the ENDF/B format, Lawrence Livermore Laboratory, Livermore, CA, UCRL-50400, Vol. 17, Part C, 1979.
- [8] D. E. Cullen, "Program SIGMA1(Version 79-1): Doppler Broaden Evaluated Cross Sections in the Evaluated Nuclear Data File/Version B (ENDF/B) Format," Lawrence Livermore Laboratory, Livermore, CA, UCRL-50400, Vol. 17, Part B, Rev. 2, 1979.
- [9] M. B. Chadwick et al., ENDF/B-VII.0: Next Generation Evaluated Nuclear Data Library for Nuclear Science and Technology, Nuclear Data Sheets, Vol. 107, p. 2931, 2006.
- [10] K. Shibata et al., Japanese Evaluated Nuclear Data Library Version 3 Revision-3: JENDL-3.3, Journal of Nuclear Science and Technology, Vol. 39, p. 1125, 2002.
- [11] A. Koning, R. Forrest, M. Kellett, R. Mills, H. Henrikson and Y. Rugama, The JEFF-3.1 Nuclear Data Library, JEFF Report 21, 2006.