

# Characteristics Analysis of Double Wall Tube Steam Generator

\* , \* , \* , \*\*  
\* , \*\*

150

-  
가 .

DWTSG

가

MONJU ,

DWTSG

MONJU

## Abstract

The DWTSG concept is proposed as a new liquid metal reactor concept to avoid sodium-water reaction incidents in the SG. This present paper attempts to assess thermal characteristics of DWTSG in the this alternate concepts. The concept provides double barrier between the sodium and the steam. This will result in improved reliability of SG compared to the conventional steam generator design using single wall tubes. The gap between the inner and outer tubes affects the heat transfer from sodium to steam. To estimate heat transfer performance of the concept, the required heat transfer area was calculated, and compared with MONJU SG. In the case of eliminating of IHTS the required heat transfer area for DWTSG is equally mated to that of MONJU design.

1.

가

-

가

1,2,3,4

(Double Wall Tube Steam Generator :DWTSG)

가

. DWTSG EBR-II

80

가

가

가,

가

CRIEPI

DWTSG

5

DWTSG

6

DWTSG

가

, DWTSG

가

## 2. DWTSG

### 2.1.

( 1.(a) ).

가

가

. DWTSG

가 가

DWTSG

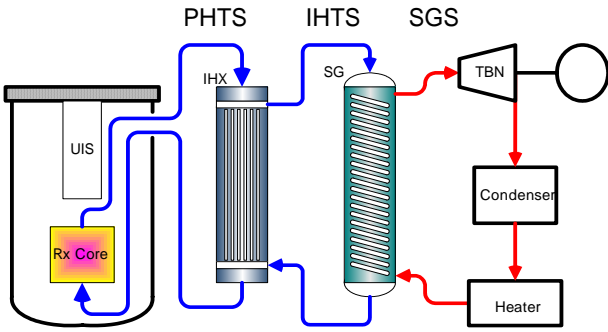
/

( 1.(b) ).

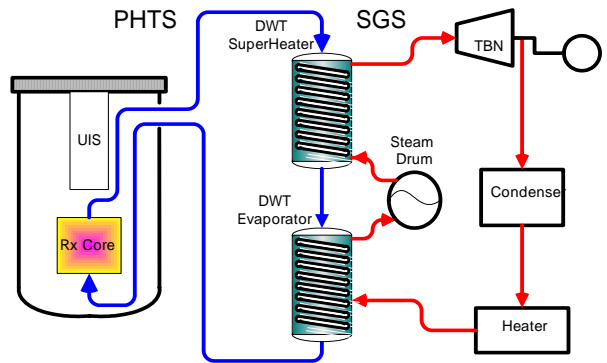
(500°C )

Sulzer

DNB



(a) Conventional LMFBR



(b) DWTSG

1

2.2.

가

가

가

(Helical coil)

1m/s

가

DWTSG

5,7,8

0.006mm~0.2mm

0.2mm

Pb-Bi

wire spacer

0.01mm

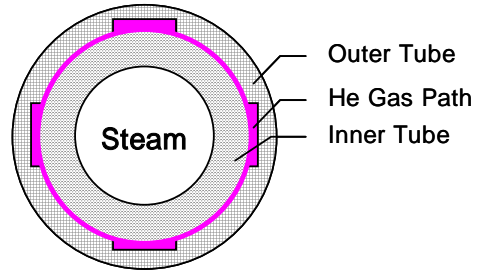
(pre-stressed)

0.01mm

4 가

1

	, mm	24.2
,	, mm	2.5
-	, mm	0.01
		14
	, mm	52
		2.254Cr-1Mo



2

3.

3.1.

DWTSG

4가

( / ) 28,400W/m-°C Roy<sup>9</sup> ( )

3.2.

DWTSG

<sup>10</sup>

가

3

DWTSG

27~32W/m-K

0.043 W/m-K

, U,

DWTSG

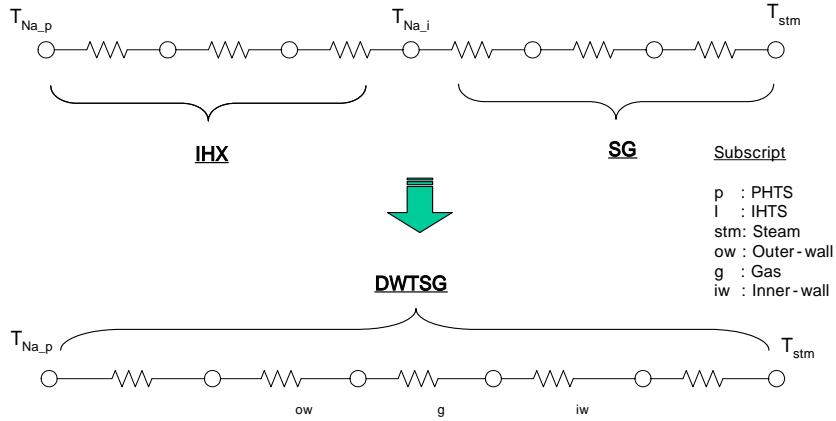
,A,

$$A = \frac{Q}{U \cdot \Delta T_{LMTD}} \quad ( \Delta T_{LMTD} : )$$

<b><u>Sodium Side</u></b>	$Nu = 4.03 + 0.028 (Re \cdot Pr)^{2/3}$
<b><u>Water Side</u></b> Subcooled Nucleate Boiling	$Nu = 0.023 Re^{0.8} \cdot Pr^{0.4}$ $h_B = S h_b + F h_c$ <p>S= Suppression factor, F: Martinelli parameter</p> $h_b = 0.00122 \left[ \frac{k_l^{.79} C_{p_l}^{.45} \rho_l^{.49}}{\sigma^{.5} \mu_l^{.29} \rho_g^{.24}} \right]^{0.68} \Delta T_{sat}^{.24} \Delta P_{sat}^{.75}$ $h_c = 0.023 \left[ \frac{k}{d_i} \right] (1-x)^{.8} Re^{.85} Pr^{.4} d_i^{.1}$
Film Boiling	$Nu = 0.0193 Re^{0.8} Pr^{1.23} \left[ x + (1-x) \frac{\rho_g}{\rho_f} \right]^{0.68} \left[ \frac{\rho_g}{\rho_f} \right]^{0.068}$
Super-heated	$Nu = 0.0073 Re^{0.936} Pr^{0.61} d_i^{0.1}$

**3 MOUJU DWTSG**

Parameters	Unit	MONJU			DWTSG	
		IHX				
Heat Capacity	MWt	238	190.8	47	190.8	47
Shell Inlet T	°C	529	469	505	502.76	529
Shell Outlet T	°C	397	325	469	397	502.76
Shell Flow rate	kg/s	1416.7	1027.8	1027.8	1416	1416
Tube inlet T	°C	325	240	367	240	367
tube Outlet T	°C	505	369	487	369	487
Tubel Flow rate	kg/s	1027.8	105.6	105.6	105.6	105.6
Steam Pressure	MPa	-	14.6	13.2	14.6	13.2



**Subscript**  
 p : PHTS  
 l : IHTS  
 stm: Steam  
 ow : Outer-wall  
 g : Gas  
 iw : Inner-wall

**3 DWTSG**

4.

DWTSG 가 MONJU  
 MONJU /  
 4 - 가  
 가 2.5mm 0.02mm DWTSG  
 MONJU /  
 DWTSG  
 0.02mm  
 2 가 1.7 가

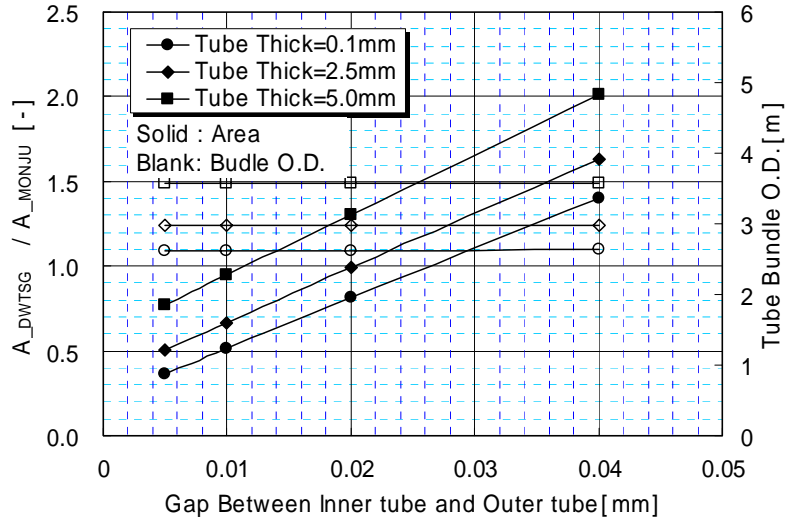
5 , DWTSG  
 (0.01mm~0.04mm) 0.7 2  
 ( 3 MONJU ) 1.5~4 가  
 DWTSG 가 ,  
 148 °C 1.6 . 0.02mm  
 DWTSG /  
 2.5

5 DWTSG

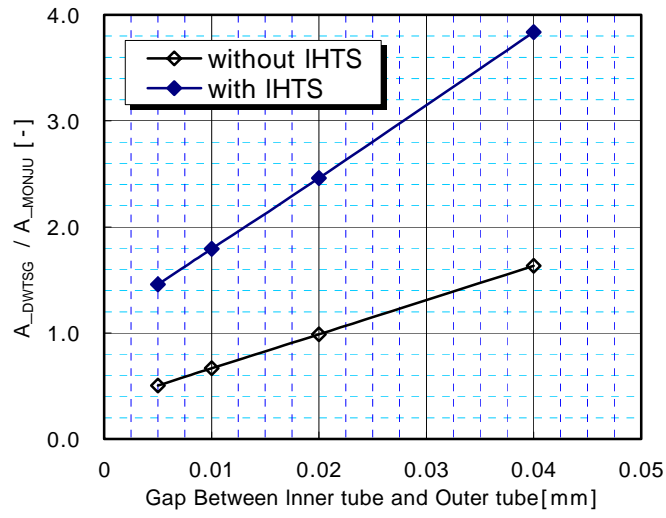
가 300 ( $k_{Pb-Bi}=14 \text{ W/m-K}$ ) Pb-Bi

DWTSG  
 0.2mm  
 40%  
 가  
 1.2 가  
 가  
 1/10  
 MONJU  
 Pb-Bi

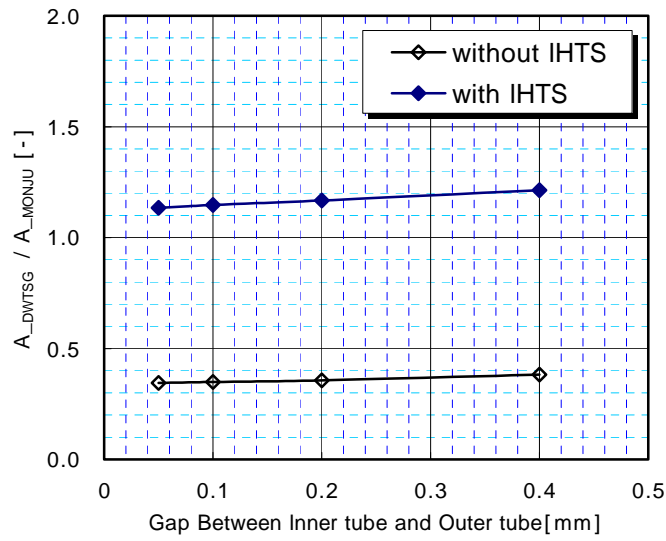
Pb-Bi  
 가  
 DWTSG 가 가  
 15% 가 Pb-Bi  
 가  
 DWTSG 가



4 DWTSG



5 /



6 - (Pb-Bi)

5.

MONJU

DWTSG  
, DWTSG

가

-



0.02mm

Pb-Bi

, DWTSG

1. Gromov, et al, "Use of Lead-Bismuth Coolant in Nuclear Reactors and Accelerator-driven System", Nuclear Engineering Design, 173, 1997
2. Seong-O Kim, Myung-hwan Wi, Yoon sub Sim, Eui kawng Kim, "Evaluation of New Design Concepts for Steam Generators in Sodium Cooled Liquid Metal Reactors", Journal of the Korea Nuclear Society, Vo1. 35, No.2, April, 2003
3. , , , "Lead-Bismuth", 2002 , 2002
4. Miyazaki et al, "Advanced IHX-SG Combined FBR System Designs and Basic Experiments" Proc. 10th Pacific Nucl. Conf. Vol.1 p769, Kobe, Japan 20-25 Oct.,1996
5. Yoshihisa Nishi, " Study on Lead-Bismuth Eutectic Utilization Technology", CRIEPI-KAERI Information Exchange on FBR Technology, Oct., 2003
6. , , , , "가", 2003 , 2003
7. L.E. Efferding et al "Preliminary Thermal Hydraulic Sizing Calculation for Duplex Tube Evaporator/Superheater", DOE/SG/00962, June, 1974
8. B.E. Dawson et al Preliminary Design : Duplex Tube Low-Pressure Saturated Steam Generator for Large LMFBR Plant", NP-1219, 1979
9. Roy, P. and Dodson, G.R., Effects of Particulate Deposition on the Heat Transfer Coefficient of LMFBR Heat Exchanger, GE Report GEFT-00265(L), 1977
10. Takeo Suzuki et al, "MONJU Secondary Heat Transport System Sodium Leak", 10th Pacific Basin Nuclear Conference, Kobe, Oct., 1997