

be obtained. An intensity of radioactivity is inversely proportional to the square of the distance. Thus, when the distance increases, the population dose is drastically reduced. It is assumed that an amount of radiation is linear between each interval because the spatial distance data of MACCS2 code is sufficiently large. The boundary is calculated using these assumptions.

2.4 Risk Assessment

The norm and basis for the safety assessment of a nuclear power plant is based on the safety goals of early fatality risk ($<5 \times 10^{-7}/RY$) and cancer fatality risk ($<2 \times 10^{-6}/RY$) which are established by the USNRC. Using the Level 3 PSA, a safety assessment of an accident scenario of a plant can be carried out. In addition, according to the General Design Criteria (GDC), when a severe accident takes place, the average release frequency of the dose exceeding 10 mSv within the exclusion area boundary (EAB) of 560m is stipulated to be below $10^{-6}/RY$. This is one of the baselines of Level 3 PSA and a safety assessment can be conducted by finding out the Complementary Cumulative Distribution Function (CCDF) of the accident scenario of the reference nuclear power plant as shown in Fig. 3 [5].

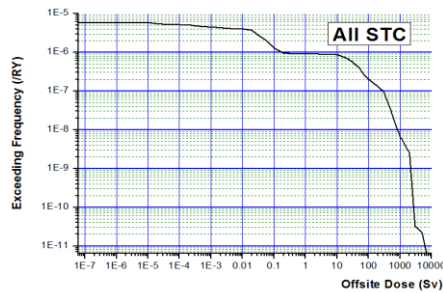


Fig.3. The CCDF of the EAB of the reference plant.

2.5 Risk Parameters

Iodine and Cesium are two major nuclides that have a great influence on human health. By obtaining the dose parameter through a sensitivity analysis on the release fraction of these two nuclides as shown in Fig.4 and Fig. 5, it is possible to assess how much a specific nuclide affects the public health directly. This method allows one to evaluate the nuclide's release, dose, and characteristics related to health effects.

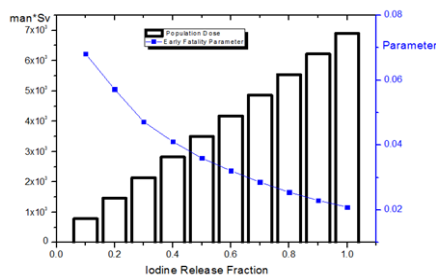


Fig.4. Parameters associated with Early Fatality for the radioactive nuclides, I.

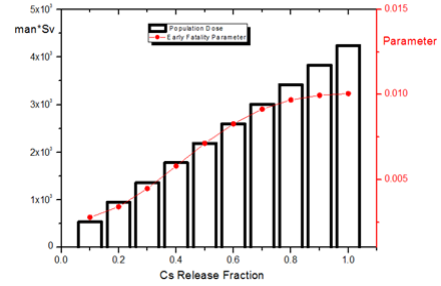


Fig.5. Parameters associated with Early Fatality for the radioactive nuclides, Cs.

3. Conclusions

A methodology for an emergency preparedness, which can be applied to evaluate the damage of the radioactive release as well as to assess the safety of the accident scenario of a nuclear power plant, has been developed and applied for the reference plants in Korea. By applying a source term analysis, an exclusive zone based on the radioactive dose is obtained. And the results of the health effect assessment based on the release fraction of specific nuclides to public with an effective emergency response activity have been simulated. A methodology utilizing the Level 3 PSA with the actual emergency response activities has been developed and applied to typical nuclear accident situations. The plausible standard for performing an emergency plan is suggested and the valuable information regarding emergency preparedness has been produced in this study. For further works, the sensitivity study on important parameters will be performed to simulate the actual severe accident situations such as sheltering, evacuation, and emergency response activities.

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