2003

,

## CANFLEX - NU 1

# Thermalhydraulic Analysis of Steam Generator Multi Tube Rupture Event for Wolsong NPP Unit 1 Loaded with CANFLEX-NU

, , , , ,

103 - 16

(CANFLEX -NU) 가 (Steam Generator Tube Rupture, SGTR) 1

1

1

가

10

SBLOCA CANFLEX -NU

### Abstracts

This study was done as part of the safety analysis for full CANFLEX-NU loaded core in Wolsong NPP unit 1. Steam generator tube rupture accident is characterized by the primary coolant leakage to steam and feed water system and fission inventory being reached directly to public bypassing containment. This accident can be devided into two cases of "Single Tube Rupture Accident" and "Multiple Tube Rupture Accident". The amount of coolant leakage in single tube rupture accident is compensated by normal  $D_2O$  feed system and thermal hydraulic behavior is remained steady state. But it is known that multiple tube rupture accident which as many as 10 tubes are failed follows thermal hydraulic behavior of small LOCA. So detailed analysis of both thermal hydraulics and trip coverage was done for multiple S/G tube rupture accident.



CANFLEX -NU .

CATHENA(cat3\_5rev1)

## 가. 가

- 1) 1 (Primary Heat Transport System): 가 (Pressurizer), (Degasser Condenser), (Degasser Cooler), (Feed & Bleed System) 가 .
- 2) 2 (Secondary Heat Transport System): (Turbine Governor Valve) (Feedwater Control Valve) 가 .
- 3) (Reactor Regulating System) : 103%
- 4) (Reactor Shutdown System): 가 .
- . 1 . 5) (Emergency Core Cooling System): (ECI), (SGCC) , ECI SGCC 가
- . 1) :
- 4 (Single Averaged Model) 1 , Nodalization 1 103%



1.	(103%)			
	1	2	3	4
(MPa(a))	11.36	11.36	11.36	11.36
( )	268	268	268	268
(MPa(a))	10.02	10.03	10.10	10.02
( )	311	311	311	311
(%)	4.8	4.6	4.7	4.8
(MPa(a))	9.57	9.58	9.57	9.57
(MPa)	1.82	1.81	1.82	1.82
(kg/s)	1901.8	1893.9	1897.3	1893.6
(MW)	527.89	527.89	527.89	527.89
(MW)	16.1			
2 (MW)	530.7	530.8	530.1	532.5
가		12	2.45	
(MPa(a))	4.69	4.69	4.69	4.69
( )	260	260	260	260
(kg/s)		107	4.76	
(kg/s)	985.37			
( )	186	186	186	186
	5.46:1	5.46:1	5.46:1	5.46:1

2)

)

:

. 06					
가	7.3MW	935kW		O6_mod	
	3		가		
	. 06		Nodalization	2	
	2				

가

	2. (103%)	
nois 7		O6_mod
Header Boundary Conditions	(103%):	
GP1-8	(MPa)	11.36
OP1-5	(MJ/kg)	2.50
	(MJ/kg)	1.13
		0.0
081-3	(MPa)	10.02
OF1-2	(MJ/kg)	2.53
UEX++ CHAN1 05561 05561	(MJ/kg)	1.35
		0.373
	(MW)	7.3
OEPU1	(kg/s)	25.07





Nodalization

2. 06



BREAK



5) (TRIP Coverage) : 25%, 50%, 75%, 100%, 103% (Liquid Zone Control), (Adjuster) (Mechanical Control Absorber) (RRS)

## 3.

- 가. 10 1 가 1 2 . (SGLC) (SGPC) 2 .
- 가 가 가 . 가 가 가 . 2 가
- . 2 . 2 (CSDV) 가. 2 CSDV

# ・ フト・・ ・ , , , ,

- · 가 (149), · ,
  - .
- 4. 가.
- 1 2 2 가 · . . . .

1) FSAR 가

,

5

,

가 100% (film boiling) . 가 600 , 가 600 ,

### 5.

•

,가 가가 (7.26m) 가 (4,5),1 ((RIH) : 11.36MPa, (ROH): 10.02MPa) ROH 1 (Low PHT Pressure Trip) 8.7MPa 3.9MPa , 가 LOCA (5.25MPa) .(6) 2 Turbine Governor Valve가 (4.7MPa) (7), 2 ( 8). ,

(ASDV) (CSDV) CSDV가 2 .( 9)













3.

( )								
0.0								
492.7	1	2	フ	<u>י</u> ት			(	)
600.0	(		)					
659.4								
663.4	1	2					(	)
697.1			(LOCA)	,		,		
705.7								
1200.1								
1600.0	(			,	15	)		
3190.0	(		가 149	,		)		

.

10 11

19kg/s 62kg/s 81kg/s가

.

,

2

가 1,2

6kg/s







O6\_mod







#### RRS\_Operating, SDS1

25% 50% 75% 100% 103%

가
1
1

RRS\_Frozen, SDS1

25% 50% 75% 100% 103%

	가
	1
	1
	1

13.

### RRS\_Operating, SDS2

25% 50% 75% 100% 103%

가	
1	

RRS\_Frozen, SDS2 25% 50% 75% 100% 103%

	가
	1
	1

6.



## 7.

- 1. 1 FSAR,
- 2. 2,3,4 FSAR,
- 3. 86-03500-AR-034 "Steam Generator Tube Rupture, wolsong NPP234", AECL, 1995
- 4. W1-CANFLEX-AR-010 "CATHENA Above Header Model (wolsong -1 CANFLEX-NU)", KOPEC, 2002
- W1 -CANFLEX AR -002 'CATHENA Trip Coverage Analysis Model (wolsong -1 CANFLEX NU)", KOPEC, 2002
- 6. W1 -CANFLEX -AR -011 'CATHENA Secondary Side Model ((wolsong -1 CANFLEX -NU)", KOPEC, 2002.
- 7. B.N.Hanna, Editor, RC -982 -4/COG -93 -140, "CATHENA Input Reference", AECL, 1992
- 8. J.P.Mallory, Editor, RC-982-5/COG-93-140"CATHENA GENHTP Input Reference", AECL,1992
- 9. A. Abdul -Razzak, et al, "CATHENA Simulations of Steam Generator Tube Rupture", AECL -Sheridan Park