

## A Conceptual Design of a Double Wall Tube Steam Generator in a Prototype Fast Reactor

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### 1. Introduction

The reaction between sodium and water is one of the most serious problems encountered in the design of a steam generator in a fast reactor. To avoid such a sodium-water reaction, a double wall tube steam generator was developed. The objective of the present study is to develop a design code (DWTSG) for a double wall tube steam generator in a fast reactor and its application to the design of a 150MWe prototype fast reactor. A one dimensional approach is employed and the related pressure loss and heat transfer correlations are presented.

### 2. Pressure Drop and Heat Transfer Correlations

#### 2.1 Pressure Drop Correlation for Sodium Side

The Blasius pressure drop correlation is used for the sodium side pressure drop;

$$\text{Re} \leq 2,100 \quad f = \frac{64}{\text{Re}} \quad (1)$$

$$2,100 \leq \text{Re} \leq 30,000 \quad f = 0.3164 \text{Re}^{-0.25} \quad (2)$$

$$\text{Re} \geq 30,000 \quad f = 0.184 \text{Re}^{-0.2} \quad (3)$$

#### 2.2 Pressure Drop Correlation for Water Side

In a single phase region, the Blasius correlation is used, and in a two-phase region, a two-phase multiplier is multiplied by the Blasius correlation. The homogeneous equilibrium model, the modified Martinelli-Nelson or Jones model and the Thom void fraction - Thom friction factor are employed for a two-phase multiplier

#### 2.3 Heat Transfer Correlation for Water Side

- Preheat region : Dittus-Boelter correlation

$$Nu = 0.023 \text{Re}^{0.8} \text{Pr}^{0.4} \quad (4)$$

- Nucleate boiling region : Chen correlation

$$h_B = S h_b + F h_c \quad (5)$$

where F is the Martinelli parameter and S is the suppression factor

$$h_c = 0.023 \left( \frac{k}{d_i} \right) (1-x)^{0.8} \text{Re}^{0.8} \text{Pr}^{0.4} \quad (6)$$

$$h_b = 0.00122 \left[ \frac{k_l^{0.79} C_{pl}^{0.45} \rho_l^{0.49}}{\sigma^{0.5} \mu_l^{0.29} h_{fg}^{0.24} \rho_g^{0.24}} \right] \Delta T_{sat}^{0.24} P_{sat}^{0.75} \quad (7)$$

- Film boiling region : Bishop correlation

$$Nu_f = 0.0193 \text{Re}_f^{0.8} \text{Pr}_f^{1.23} \left[ x + (1-x) \frac{\rho_g}{\rho_f} \right]^{-0.68} \left( \frac{\rho_g}{\rho_f} \right)^{0.068} \quad (8)$$

- Super heat region : Dittus-Boelter correlation

$$Nu = 0.023 \text{Re}^{0.8} \text{Pr}^{0.4} \quad (9)$$

-Fouling

$$h_{Fw} = 2.84 \times 10^4 W / m^2 \text{ } ^\circ C \quad (10)$$

- Gap conductance

$$h_g = 3.5 \times 10^4 W / m^2 \text{ } ^\circ C \quad (11)$$

-Critical quality : Duchatelle correlation

$$x = 1.69 \times 10^{-4} q^{0.719} G^{-0.212} e^{2.5 \times 10^{-8} P} \quad (12)$$

#### 2.4 Heat Transfer Correlation for Sodium Side

- Graber-Rieger correlation :

$$Nu = a + b(Pe)^c \quad (13)$$

$$a = 0.25 + 6.20(P/D_h) \quad (14)$$

$$b = -0.007 + 0.032(P/D_h) \quad (15)$$

$$c = 0.8 - 0.024(P/D_h) \quad (16)$$

- Lubarsky-Kaufman correlation :

$$Nu = 0.625(Pe)^{0.4} \quad (17)$$

### 3. One Dimensional Model

#### 3.1 Continuity equation

$$w_s = \text{const} \quad (18)$$

$$w_w = \text{const} \quad (19)$$

#### 3.2 Momentum equation

$$\Delta p = \Delta p_{acc,i} + \Delta p_{fric,i} + \Delta p_{grav,i} \quad (20)$$

$$\Delta p_{acc,i} = \left( \frac{G_w^2}{\rho} \right)_i - \left( \frac{G_w^2}{\rho} \right)_{i+1} \quad (21)$$

$$\Delta p_{fric,i} = \frac{L_l}{d_i} \frac{G_w^2}{2\rho_l} + f \frac{L_{2\Phi}}{d_i} \overline{\Phi}_{lo}^2 \frac{G_w^2}{2\rho_f} + f \frac{L_g}{d_i} \frac{G_w^2}{2\rho_g} \quad (22)$$

$$\Delta p_{grav,i} = \rho_l g L_l + \langle \overline{\rho_H} \rangle g L_{2\Phi} + \rho_g g L_g \quad (23)$$

$$\text{where } \langle \overline{\rho} \rangle_i = \frac{\langle \rho \rangle_i + \langle \rho \rangle_{i+1}}{2}, \quad \langle \rho \rangle_i = \frac{1}{v_f + \langle x \rangle_i v_{fg}} \quad (24)$$

### 3.3 Energy equation

-Heat transfer from the tube wall :

$$\Delta Q = U \Delta A_o \Delta T_o \quad (25)$$

-Heat transfer from sodium flow :

$$\Delta Q = w_s (h_{s,in} - h_{s,out}) \quad (26)$$

-Heat transfer from water/vapor

$$\Delta Q = w_w (h_{w,out} - h_{w,in}) \quad (27)$$

$$\text{where } \Delta T_o = \frac{(T_{s,in} + T_{s,out})}{2} - \frac{(T_{t,in} + T_{t,out})}{2} \quad (28)$$

$$\Delta A_o = \pi d_o L \quad (29)$$

- Overall heat transfer

$$\begin{aligned} \Delta Q &= h_s \Delta A_o (T_s - T_{Fs}) = h_{Fs} \Delta A_o (T_{Fs} - T_o) \\ &= \frac{2k}{d_o \ln(\frac{d_o}{d_g})} \Delta A_o (T_o - T_{g+}) = h_g \Delta A_g (T_{g+} - T_{g-}) \\ &= \frac{2k}{d_g \ln(\frac{d_g}{d_i})} \Delta A_g (T_{g-} - T_i) = h_{Fw} \Delta A_i (T_i - T_{Fw}) \\ &= h_w \Delta A_i (T_{Fw} - T_w) \end{aligned} \quad (30)$$

$$U = \frac{1}{\frac{1}{h_s} + \frac{1}{h_{Fs}} + \frac{d_o}{2k} \ln(\frac{d_o}{d_i}) + \frac{d_o}{d_g} \frac{1}{h_g} + \frac{d_o}{d_i} (\frac{1}{h_{Fw}} + \frac{1}{h_w})} \quad (31)$$

$$T_o = T_s - \frac{\Delta Q}{\Delta A_o} \left( \frac{1}{h_s} + \frac{1}{h_{Fs}} \right) \quad (32)$$

$$T_i = T_w + \frac{\Delta Q}{\Delta A_i} \left( \frac{1}{h_{Fw}} + \frac{1}{h_w} \right) \quad (33)$$

## 4. Results and Discussion

The computer code DWTSG is developed for the design of a double wall tube steam generator in a 150MWe prototype fast reactor [1]. From the test design, it is shown that the present code can be used in a practical design [2]. Table-1 shows the operating

conditions and design results for the prototype fast reactor being developed at KAERI.

Table-2 Design Results

SG type	Straight double wall tube
Thermal capacity (MWT)	196.8
Number of tubes (without plugged tubes)	704 (640)
Sodium inlet/outlet temperature (°C)	527/324
Feed water inlet/steam outlet temperature (°C)	230/503
Sodium/Feed water flow rate (kg/s)	757.95/85.35
Feed water inlet/steam outlet pressure (MPa)	19.5/16.5
Tube heat transfer length (m)	28
Tube inner/gap/outer diameter (mm)	13.8/16/19
Tube inner/outer thickness (mm)	1.1/1.5
Heat transfer area (m <sup>2</sup> )	1068
SG tube bundle diameter (m)	0.885
Tube longitudinal and transverse P/D	1.67
Tube material	Mod 9Cr-1Mo
SG height (m)	~36
Tube side pressure loss (kPa)	192
Shell side pressure loss (kPa)	234
Distance between bundle and shell (m)	~0.0157
SG shell inner diameter (m)	0.954

## 5. Conclusions

A design code for a double wall tube steam generator (DWTSG) in a fast reactor was developed. The theoretical basis is briefly presented and the correlations for pressure loss and heat transfer are presented. Theoretically, a one dimensional model is used and is briefly summarized. The code is applied to the design of the double wall tube steam generator in a 150MWe prototype reactor and the design results are presented.

## ACKNOWLEDGEMENTS

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## REFERENCES

- [1] S. K. Choi, User Manual for DWTSG Computer Code, LMR/FS200-CM-01-Rev.0/07, 2007.
- [2] S. K. Choi, Thermal Sizing of a Double Wall Tube Steam Generator for a Prototype Fast Reactor, SFR-FS400-CA-02-2012Rev.01, 2012.