

Fabrication of Micro-cell UO₂ Pellet for HALDEN Irradiation Test

Dong-Joo Kim*, Keon Sik Kim, Jong Hun Kim, Young Woo Rhee, Jang Soo Oh, Jae Ho Yang, Yang-Hyun Koo

LWR Fuel Technology Division, Korea Atomic Energy Research Institute,
989-111 Daedeok-daero, Yuseong, Daejeon 34057, Rep. of Korea

*Corresponding author: djkim@kaeri.re.kr

1. Introduction

A micro-cell UO₂ pellet is being developed in KAERI (Korea Atomic Energy Research Institute) to enhance the accident tolerance of nuclear fuels under accident conditions as well as the fuel performance under normal operation conditions [1].

The micro-cell UO₂ pellet consists of UO₂ grains or granules enveloped by thin cell walls. Depending on the materials used for making the cell walls, there are ceramic and metallic micro-cell UO₂ pellets.

The ceramic wall in ceramic micro-cell UO₂ pellets is composed of oxides having chemical affinity to volatile fission products such as Cs or I, which are highly radioactive and corrosive fission products, and act as multiple traps to immobilize the volatile fission products. That is to say, the ceramic micro-cell walls can block the migration of fission products to the pellet outside. The increased retention capability of fission products will reduce the stress corrosion cracking at the inner surface of cladding as well as the rod internal pressure.

By implementing the metallic cell walls with high thermal conductivity, the thermal conductivity of a micro-cell UO₂ pellet can be increased. The mobility of the fission gases is reduced by the lower temperature gradient in the UO₂ fuel pellet. That is, the capability of

the fission product retention of the fuel pellet will increase. In addition, the micro-cell UO₂ pellet with high thermal conductivity will significantly increase the safety margin under design basis accidents such as a loss-of-coolant accident (LOCA) as well as the thermal margin under normal operation conditions [2].

In the development of nuclear fuel, the irradiation behavior of the developed fuel material must be considered based on various aspects (pellet microstructure, cell wall soundness, cell wall material behavior, pellet structural integrity, etc.). Above all, it is important that the designed in-reactor fuel performances of the micro-cell UO₂ pellet are verified.

To investigate the irradiation behaviors of the micro-cell UO₂ fuel pellet materials, a HALDEN irradiation test is planned for two kinds of micro-cell UO₂ pellets. This paper reports the specifications of the fabricated micro-cell UO₂ pellets for the irradiation test.

2. Irradiation Test Plan and Pellet Specifications

Through the cooperation with Thor Energy, a HALDEN irradiation test of the micro-cell UO₂ pellets will be performed. KAERI is participating in the International Thorium Consortium organized by Thor Energy. In the consortium, several consortium members (the Institute for Transuranium Elements (ITU), Germany; Fortum, Finland; National Nuclear Laboratory (NNL), U.K.; Westinghouse, U.S.) are participating.

The irradiation test rigs with instrumentations will be manufactured at the Institute for Energiteknikk (IFE), Norway. Table 1 shows brief information for the irradiation test.

Two kinds (ceramic and metallic) of micro-cell UO₂ pellets were fabricated [3, 4] in accordance with the requirements for the HALDEN irradiation test. The number of samples (~60 EA) was sufficiently provided to manufacturing the instrumented rigs. The prepared pellet specifications are shown in Table 2.

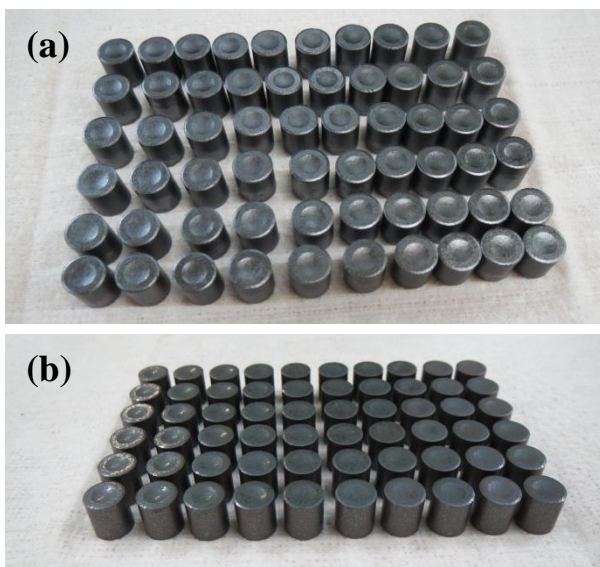


Fig. 1. Two kinds of fabricated samples for irradiation test; (a) ceramic and (b) metallic micro-cell UO₂ pellets.

Table 1. Brief information for the irradiation test

Irradiation period	2015. 10. - 2017. 9.
Estimated burnup	~25 GWd/mtU
Test rig # in HALDEN reactor	IFA-790

Table 2. Specifications of prepared pellet samples

	Ceramic micro-cell pellet	Metallic micro-cell pellet
Cell wall materials	0.6 wt% Si-Ti-O (2 vol%)	3.4 wt% Cr (5 vol%)
Averaged cell size* (μm)	~80	~290
Pellet density (g/cc)	10.73 \pm 0.03	10.45 \pm 0.03
Pellet diameter** (mm)	8.190 \pm 0.002	8.196 \pm 0.001
Pellet height (mm)	9.4 \pm 0.2	9.12 \pm 0.03
Pellet weight (g)	5.15 \pm 0.10	4.93 \pm 0.02
U enrichment	4.5%-U ²³⁵	4.5%-U ²³⁵

* A ceramic micro-cell consists of one grain. Therefore, the cell size of a ceramic micro-cell pellet is equal to the grain size. On the other hand, a metallic micro-cell consists of a lot of grains (6-7 μm).

** Pellet diameter after centerless grinding process.

[4] D.J Kim, Y.W. Rhee, J.H. Kim, K.S. Kim, J.S. Oh, J.H. Yang, Y.H. Koo, K.W. Song, Fabrication of micro-cell UO₂-Mo pellet with enhanced thermal conductivity, Journal of Nuclear Materials, Vol.462, p.289, 2015.

3. Summary

To investigate the irradiation behaviors of the micro-cell UO₂ fuel pellet materials, a HALDEN irradiation test is planned for two kinds of micro-cell UO₂ pellets. Two kinds (ceramic and metallic) of micro-cell UO₂ pellets were prepared.

The in-situ data of irradiated micro-cell UO₂ pellets are expected to be obtained, and the progress of the irradiation testing continuously reported. Through the irradiation test and post-irradiation examination, the designed fuel performances of the micro-cell UO₂ fuel pellets will be verified.

ACKNOWLEDGEMENT

This work was supported by the National Research Foundation of Korea (NRF) grant funded by the Korea government (MSIP) (No. 2012M2A8A5025821).

REFERENCES

- [1] Y.H. Koo, J.H. Yang, J.Y. Park, K.S. Kim, H.G. Kim, D.J. Kim, Y.I. Jung, K.W. Song, KAERI's development of LWR accident-tolerant fuel, Journal of Nuclear Technology, Vol.186, p.295, 2014.
- [2] K.A. Terrani, D. Wang, L.J. Ott, R.O. Montgomery, The effect of fuel thermal conductivity on the behavior of LWR cores during loss-of-coolant accidents, Journal of Nuclear Materials, Vol.448, p.512, 2014.
- [3] D.J. Kim, Y.W. Rhee, J.H. Kim, K.S. Kim, J.S. Oh, J.H. Yang, Y.H. Koo, Optimization of UO₂ granule characteristics for UO₂-Mo pellet fabrication, the Korean Nuclear Society Spring Meeting, Jeju, Korea, May 29-30, 2014.