# Comparison between Alloy 600 and Alloy 690 ODSCC behavior

Dong-Jin Kim, Hong Pyo Kim and Seong Sik Hwang

Nuclear Materials Safety Research Division, Korea Atomic Energy Research Institute(KAERI), Yuseong, Daejeon,

Korea, 305-353

<sup>\*</sup>Corresponding author: djink@kaeri.re.kr

## 1. Introduction

Stress corrosion cracking (SCC) has been occurred on Alloy 600 steam generator (SG) tubes frequently in the past and is still being occurred.

Many efforts to reduce SCC degradation were tried and then SCC resistance was improved by applying a heat treatment process from LTMA (low temperature mill annealed) Alloy 600 to HTMA (high temperature mill annealed) Alloy 600, and then TT (thermally treated) Alloy 600.

Intra-granular carbide widely spread in LTMA Alloy 600 dissolves, and inter-granular carbide is then formed during high-temperature mill annealing and cooling, which leads to a great SCC resistance enhancement. Inter-granular carbide is well developed, healing chromium depletion at a grain boundary, and residual stress is removed during additional thermal treatment following mill annealing, which improves the SCC resistance more [1].

In spite of this improvement of Alloy 600TT, Seabrook and Vogtle 1 in the US, using Alloy 600TT, also showed SCC due to a non-optimum microstructure, residual stress, Pb existence, and so on over a 20-year operation of an NPP even though SCC occurs less frequently than LTMA and (or) HTMA Alloy 600s [2,3]. SCC has also occurred for Alloy 600TT tubes in Korea, whose main causes resemble US cases. Hence, SCC on Alloy 600 should overcome because many plants use Alloy 600 as SG tubes.

Contrary to Alloy 600, Alloy 690 as an alternative of steam generator tubing material for Alloy 600 shows an excellent SCC resistance in most environments. Steam generator tubing materials are being selected as Alloy 690 in newly constructed nuclear power plants and replaced steam generators. There is no report about ODSCC (outside diameter SCC) for Alloy 690 which was first used in NPP (nuclear power plant) in 1988.

However it was reported [4,5] that Alloy 690 is not resistant to SCC in an alkaline environment including the pH range attainable in the SG crevice and moreover more susceptible to SCC in leaded alkaline solution.

In the present work, SCC resistance of Alloy 690 was compared with that of Alloy 600 in an alkaline solution based on SCC test results, which was discussed in terms of passivity, chemical stability and alloying element. Environment weighted improvement factor for Alloy 690 was also introduced.

### 2. Experimental

The slow strain rate tension (SSRT) test was performed for uniaxial tension specimens fabricated from the nickel based alloys in the unleaded and leaded solutions. The tests were carried out in half gallon nickel autoclaves at 315 °C and an equilibrium pressure. The test specimens were immersed at an open circuit potential (OCP) without an impressed electrochemical current. The strain rate was  $2 \times 10^{-7} \text{ s}^{-1}$ . Test procedure was reported elsewhere [5].

RUB (reverse u bend) samples were fabricated using a mandrel, bolt, nut and washer. The radius of RUB was 12.975mm, which was comparable to the previous report of 12.5mm [6]. According to the previous report [7], it was reported that the stress level of UTS (ultimate tensile stress) is applied to the apex of the RUB specimen in a RUB specimen radius range of 9-13mm.

The test specimens were immersed in 3.78 L Ti autoclaves. The pH (T) of the solution is expected to be 9.5 at  $310^{\circ}$ C by adjusting the NaOH concentration in a solution containing 3m NaCl + 500ppm Pb.

The immersion test was carried out for rectangular plate specimens (10 mm x 10 mm) fabricated from the tubing. The surface of the specimens was polished up to 1  $\mu$ m using a diamond suspension. A nickel wire was spot welded to the specimen, and the wire was shielded with a heat-shrinkable polytetrafluoroethylene (PTFE) tubing. The immersion test was performed in a 1-gallon nickel autoclave at 315 °C for 28 days.

Materials used for the test were Alloy 600 HTMA, Alloy 600TT and Alloy 690TT, which were mill annealed at 1024°C for 3 min after solution annealing, thermally treated at 704 °C for 15 h after solution annealing at 975 °C for 20 min and thermally treated at 715°C for 10 h after solution annealing at 1105°C for 2 min, respectively.

Various alkaline solutions were made using highpurity water [room temperature resistivity of  $18M\Omega$ •cm]. The surface of the specimens for the immersion test was polished up to a 1 µm diamond suspension. Reagent grade PbO was added to the 0.1M NaOH solution as a source of lead.

Deaeration for SSRT and immersion tests was accomplished through high purity nitrogen gas purging for 20 h. For RUB test, the hydrogen concentration of 6ppm and deoxygenation for all autoclave tests was achieved at room temperature by three pressurizing/aspirating cycles from a 100psia to 1000psia overpressure of a 10% hydrogen/90% argon mixture gas and heating with a 200psia overpressure of 5% hydrogen/95% argon mixture gas, based on a previous report [6].

During the immersion test, an electrochemical impedance measurement was performed at the opencircuit potential with a perturbation level of 10mV in the frequency range of  $10^6$  to  $10^{-3}$  Hz using a Solartron 1260 frequency response analyser connected with a Solartron 1287 electrochemical interface for a 3 electrode system. A Nickel wire was spot welded to the specimen as the working electrode, and the wire was shielded with PTFE tubing. The nickel and Pt wires were used as a reference electrode and counter electrode, respectively.

After the immersion test, the surface oxide morphology was observed using SEM (JEOL JSM-6300). The surface oxide layer and its composition for the TEM samples prepared by FIB were examined using a FE TEM-EDS.

#### 3. Results and discussion

SCC susceptibility for Alloy 690TT was increased with the pH(T) higher than 10.2 and aggravated by adding PbO. However SCC susceptibility was decreased in highly caustic solution of 10.9 of pH(T) as shown in Fig.1(a).

Unlike Alloy 690TT, SCC resistance of Alloy 600MA was stronger than Alloy 690TT in highly caustic solution. Rather, Alloy 600MA showed SCC susceptibility in mild alkaline solution and aggravated susceptibility by adding PbO. (Fig. 1(b))

Based on SSRT test results, factor of improvement (FOI) for Alloy 690, compared with Alloy 600 can be determined as follows. The results were shown in Table 1.

- $\blacksquare$  SCC rate = length / time
- **Time to a certain length,** t = 1 / SCC rate
- FOI of Alloy 690 = t(Alloy 690) / t(Alloy 600)= SCC rate (Alloy 600) / SCC rate (Alloy 690)

According to the RUB test in a caustic solution including PbO and chloride of pH(T) of 9.5, Alloy 690TT showed significant SCC susceptibility while Alloy 600MA and Alloy 600TT showed better SCC resistance as shown in Table 2.

Fig. 2 shows environment weighted improvement factor for Alloy 690 versus Alloy 600MA which was first proposed by EPRI [8]. They considered weight factors of oxidizing environment, lead contamination, chloride, sulfur and caustic condition. From this curve, integrity or weakness of Alloy 690 can be assessed, compared with Alloy 600 exposed to the same environment. Unfortunately, very limited experimental data were used to establish environment weighted FOI because of insufficient data base under various environments.

It is found that chloride shows synergic effect on SCC susceptibility with lead according to recent result at this

laboratory while the chloride effect on SCC in caustic solution was ignored determining the original environment weighted FOI. The same FOI for chloride as the FOI in alkaline solution without chloride was used in the alkaline pH range to determine overall environment weighted FOI curve, which seems to cause over estimation of FOI.

Based on additional experimental results under various conditions such as the effect of the only chloride on SCC, FOI curve needs to be modified to apply to SG tubing material integrity assessment.

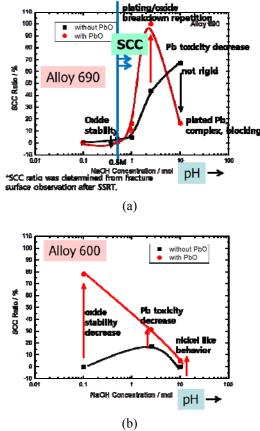


Fig. 1. SCC ratio of (a) Alloy 690TT and (b) Alloy 600MA as a function of the pH.

Table 1. SSRT results and FOI of Alloy 690.

MF 14月、 あとう											
	n al con	Denstin	FLIG (TER) Kanati	SC ale		Belander (K. ander Bellik) (Br. 4 Ben (K) (B)	iktor of inprovement (Filey 1986)	-			
ANDER		e	2	<b>2</b>	125.5	6.000-00	ille 955 165 en nafte deleminei	N (922)=00			
A65011				0	800.75	0.008 - 00					
ACCESSION OF	9.21	c	•	843	Desert	4981-92	1421	Supply RCaf disperts, July 200 to one another to			
A00011				1.4	681	7261-09		VALUE in Americal science			
ANDER			0	402	42.0	199548	517	NX decrements new descentering inform (itse 600 is			
ANXIII	[ ]			1.4	8/5	7,239-09		soluterately effected by Fig. 4 by 420 h one strong			
JARSHI.		0	x	773	1.11	32666	624	1982)-Shi, Kite Sénagébia ta SCC tim Alay Bi			
AGROTT	394			415	412	1,401 07					
A9800-		٩	•	21	55	1256	6633	Fil is mandemant in stilling Fil			
A690TT				99.9	52.37	2,655.06					
								1 1			
AGONG.	- 204	٩	×	2	688.6	0.005-00	0.0	si (625)-664, Alby 400 & robiliziy sharay in 304			
ASSOTT				67.1	144	6.17E 07		(ed)			
AMERA		a	0		446	1000		Conservativity 254 InGI+FNO. (Ibr GD income.			
ANXIII				16.5	120	4./2~05		letter			

Table 2. RUB test results and FOI of Alloy 690.

ana (and fe, az vening) Analon – 115											
<b>Methoda</b>	14	the last	in the		a di	SCC and (com/4)	Refer of Improvement Solary 1985	•			
AGOUTT	64in	3m	2299977	fore. Annelles	ы	2.50540 A 117 10	0.13	Hard International State of the International State			
A69011						1.18E-08	-	anisten franzöhren (d. 1997). Historia (d. 1997), and die anderstander (d. 1997).			
in to dive . Month											

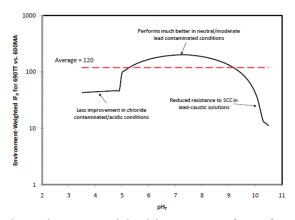


Fig. 2. Environment weighted improvement factor for Alloy 690TT versus Alloy 600MA.

Thick Oxygen Affected Layer (OAL) was observed after immersion test for Alloy 690 unlike Alloy 600 as shown in Fig. 3, which is similar to penetrative oxide observed in primary side. The OAL was increased in thickness with the immersion time. It was revealed that this layer was distributed as alternative oxide and metallic laver.

This indicates that electrolyte can penetrate into deep inside. It is also presumed that fast oxidation of Cr induces Cr depletion within very narrow region (Ni enrichment). This layer can be potential threat to brittle cracking of Alloy 690. The OAL was discussed in view of oxidation kinetics and thermodynamics.

SCC susceptibility for Alloy 690TT was increased with the pH(T) higher than 10.2 and aggravated by adding PbO. However SCC susceptibility was decreased in highly caustic solution of 10.9 of pH(T).

Alloy 690TT showed significant SCC susceptibility in combined caustic solution including PbO and chloride of pH(T) of 9.5. Unlike Alloy 690TT, SCC resistance of Alloy 600MA was stronger than Alloy 690TT in highly caustic solution. The difference in SCC behavior was expressed in terms of passivity, chemical stability and alloving element.

Environment weighted FOI for Alloy 690 versus Alloy 600 need to be revised based on accurate and wide data base.

Oxygen affected layer was observed and its thickness was increased with the immersion time for Alloy 690TT, which was discussed in view of oxidation kinetics and thermodynamics. This layer can be threat to brittle cracking.

#### REFERENCES

- [1] H. Feldman, 1018887, EPRI report, Palo Alto (2009).
- [2] S. Choi, 1020993, EPRI report, Palo Alto (2010).
  [3] J. Benson, presented at 29<sup>th</sup> annual EPRI steam generator NDE workshop, July 12-14, 2010 Vail, Colorado.

[4] R. W. Staehle, Proc. of 11th Int. Symp. on Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors, Stevenson, WA, Aug. 10-14, 2003, p. 381.

[5] D. J. Kim, H. P. Kim and S. S. Hwang, Nuclear Engineering Tech., 45 (2013) 67.

[6] Effects of Alumino Silicate on the Stress Corrosion Cracking of Alloys 600 and 690, EPRI, Palo Alto, CA: 2002. 1002782.

[7] C.O.Ruud, D.J. Snoha and A.R. Mcllree, Experimental mechanics, 1989, p. 54.

[8] S. Choi, 1019044, EPRI report, Palo Alto (2009).

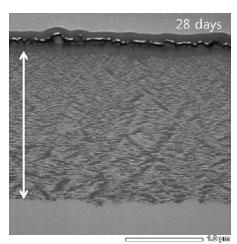


Fig. 3. TEM image for Alloy 690TT cross section after immersion test in 0.1M NaOH + 1,000 ppm PbO during 28 days.

### 4. Conclusion

ODSCC behavior of Alloy 690 and Alloy 600 as a SG tubing material was compared using SSRT and RUB test results performed at 310 and 315°C in various alkaline solutions with and without PbO.