### An Evaluation of Eigenvalue Uncertainty Caused by Monte Carlo Uncertainties of Multi-group Cross Section

### 2016 KNS Jeju, May 13<sup>th</sup> 2016 Speaker: Jae Yong LEE(HYU)





# Outline

### Introduction

- Motivation
- Goal

### Method

- Monte Carlo Method to Calculate Multi-group Cross Section
- Error Propagation
- Correlated Sampling Method

### Result

- Problem 1
- Problem 2

### Conclusion



# Introduction

### **Reactor Analysis**

- Whole Core Calculation
- FGC is Calculated from Fine Group Xs
- Fine Group Xs

Detailed in Geometry, Detailed Energy

 Accurate Calculations Required
 Uncertainty of fine group Xs may occur significant biases in next steps

#### **Modern Reactor Analysis Scheme**





# Introduction

- **Consider Complicate Geometry Pin cell** •
  - VHTR TRISO particle Fuel, Research Reactor
- **Group XS Generation for Complicate Geometry** 
  - Monte Carlo(MC) Method can be advantageous
- **Monte Carlo Method Properties** 
  - Geometry Description

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- Result with Uncertainty  $\sigma$
- Computationally Intense:  $\frac{1}{\sigma} \sim N^2$
- Potentially Accurate



**Complicate Geometry** 

#### Modeling

#### Uncertainty





3 cm

# Introduction

• Group Xs Generated via MC

Each Group Xs has uncertainties.

- MC Uncertainties causes biases in Results
   Fine Group Xs is input of next calculation
   Steps, uncertainty of Group Xs may occur
   biases in results.
- Potentially Accurate

More computational load, Better result

• Goal of This Study

Evaluate Uncertainty of k-eff caused by MC uncertainty of group Xs.





# METHOD





#### **Group XS and Uncertainty**

• Group Averaged Xs

$$-\sigma_{g} = \frac{\int_{V} \int_{E_{g}} \int_{4\pi} \sigma(r, E) \phi(r, E, \Omega) d\Omega dE dr}{\int_{V} \int_{E_{g}} \int_{4\pi} \phi(r, E, \Omega) d\Omega dE dr}$$

• Track Length Estimator

$$-\sigma_g = \frac{\sum_i \sigma_i W_i L_{g,i}}{\sum_i W_i L_{g,i}}$$

- Uncertainty of group Xs
  - Correlation between to variables

$$- \bar{x} = \frac{\bar{u}}{\bar{v}}, r_{\bar{x}} = \sqrt{\frac{\Sigma u_i^2}{(\Sigma u_i)^2} + \frac{\Sigma v_i^2}{(\Sigma v_i)^2} - \frac{2\Sigma u_i v_i}{\Sigma u_i \Sigma v_i}}$$
$$- \sigma(\bar{x}) = \bar{x} \times r_{\bar{x}}$$

• SUIT Code (HYU)

#### Part of u-235 Total Xs (81g)





#### **Uncertainty Propagation**

• Assume eigenvalue is function of each group cross section

$$k_{eff} = f(Xs_{g1}, Xs_{g2}, Xs_{g3}, \dots, Xs_{gn})$$

• Uncertainty of eigenvalue can be written as follow:

$$\sigma(k_{eff})^{2} \approx \frac{\partial^{2}k_{eff}}{\partial X s_{g1}^{2}} \sigma(X s_{g1})^{2} + \frac{\partial k_{eff}}{\partial X s_{g1}} \frac{\partial k_{eff}}{\partial X s_{g2}} cov(X s_{g1}, X s_{g2}) + \frac{\partial k_{eff}}{\partial X s_{g1}} \frac{\partial k_{eff}}{\partial X s_{g3}} cov(X s_{g1}, X s_{g3}) + \cdots$$

• It simply written as matrix form:

$$\sigma \big( k_{eff} \big)^2 \approx J V J^T$$

where 
$$J = \left(\frac{\partial k_{eff}}{\partial X s_{g1}}, \frac{\partial k_{eff}}{\partial X s_{g2}}, \dots, \frac{\partial k_{eff}}{\partial X s_{gn}}\right), \quad V(i,j) = cov(X s_{g,i}, X s_{g,j})$$



#### **Group XS and Uncertainty and Covariance**

• 
$$\sigma(k_{eff})^2 \approx JVJ^T$$

• 
$$V =$$

$$\begin{pmatrix} \sigma_{xs_1}^2 & \cdots & cov(xs_1, xs_n) \\ \vdots & \ddots & \vdots \\ cov(xs_n, xs_1) & \cdots & \sigma_{xs_n}^2 \end{pmatrix}$$

• **Cov**(**i**, **j**) = **Corr**(**i**, **j**) × 
$$\sigma_i \times \sigma_j$$

- Union Tally
- Assume  $Corr_{flux} \approx Corr_{xs}$



• 
$$\sigma(k_{eff})^2 \approx JVJ^T$$
  
 $J = (\frac{\partial k_{eff}}{\partial X s_{g_1}}, \frac{\partial k_{eff}}{\partial X s_{g_2}}, \dots, \frac{\partial k_{eff}}{\partial X s_{g_n}})$ 

#### **Correlated Sampling Method**



• Storing perturbed weights, Several results can be calculated by one input



$$\sigma(k_{eff})^2 \approx J V J^T$$

- V: MC Calculation to Generate XS
- J : Correlated Sampling Method





#### Prediction

#### Validation





# RESULT





#### Godiva



Material	Uranium(5.27 u235 w/o)
Density	18.74g/cc
Number of Particle	3000



•  $\sigma_{pr}(k_{eff}) \approx 42 \ pcm$ 



Godiva **100 Calculations** 1.0055 8.741cm 1.0050 1.0045 k-effective 1.0040 Material Uranium(5.27 u235 w/o) Density 18.74g/cc 1.0035 Number of Particle 3000 1.0030 20 40 60 80 100 0

•  $\sigma_{pr}(k_{eff}) \approx 42 \ pcm$ 



Number of Calculation

#### **U-Zr Fuel Pin cell**



Fuel Material	U-Zr (u-235 19.75w/o)
Density	7.86 g/cc
Number of Particle	3750







• 
$$\sigma_{pr}(k_{eff}) \approx 330 \, pcm$$



# CONCLUSION





# Conclusion

### **Goal of this study**

- Evaluate Multiplication Uncertainty caused by MC uncertainty of Group Xs

### **Used Method**

- Monte Carlo Method to Calculate Multi-group Cross Section
- Correlated Sampling Method
- Uncertainty Propagation

### Conclusion

- Using methods introduced in this study, it is possible to guess uncertainty of eigenvalue caused by monte Carlo uncertainty of group cross section.
- Using this study, it is expected that efficient calculation can be performed to generate Xs via MC method

