

Experimental Study of Natural Circulation in CLOF (Complete Loss Of RCS Flow rate) Accident with SMART-ITL

Jin-Hwa Yang^{*}, Hwang Bae, Sung-Uk Ryu, Byong Guk Jeon, Hyun-Sik Park

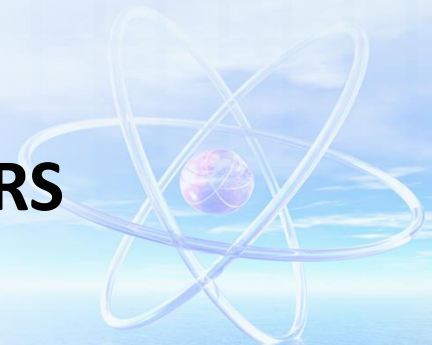
Korea Atomic Energy Research Institute

2017. 05. 19

Contents

- **Introduction: SMATR-ITL & PRHRS**
- **Experiment: CLOF Test**
- **Results & Analysis**
- **Conclusion & Further Study**

1. INTRODUCTION: SMATR-ITL & PRHRS

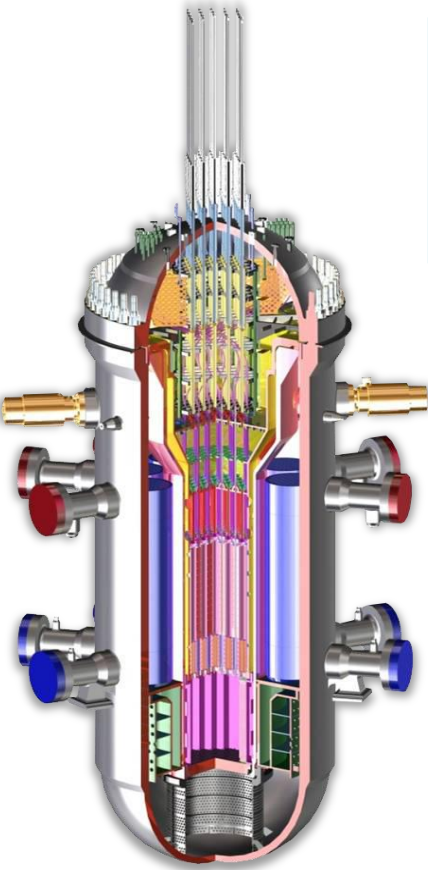


2. Experiment: CLOF Test
3. Results & Analysis
4. Conclusion & Further Study

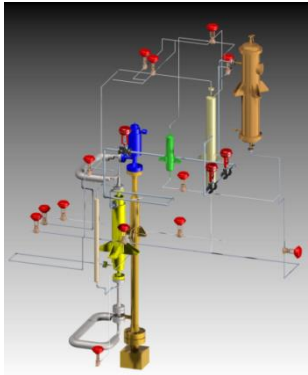
SMART-ITL (Integral Test Loop)

❖ Scaling Methods

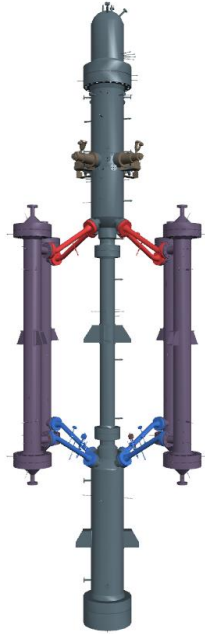
Volume scaling
(main components)
+
Three-level scaling
(partially)



SMART



***VISTA-ITL**



**SMART-ITL
(**FESTA)**

*Experimental Verification by
Integral Simulation of Transient and Accident

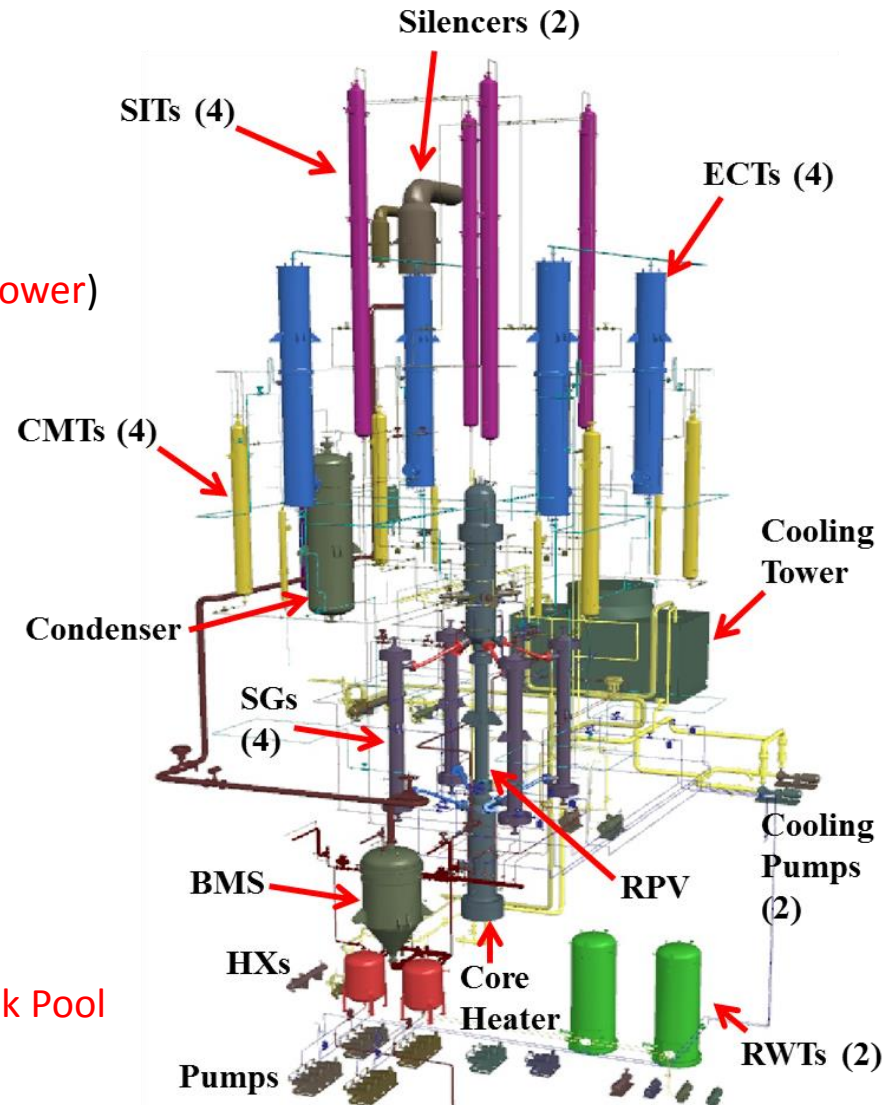
Parameters	Scale Ratio	**FESTA	*VISTA-ITL
Length, l_{OR}	l_{OR}	1/1	1/2.77
Diameter, d_{OR}	d_{OR}	1/7	1/21.75
Area, a_{OR}	d_{OR}^2	1/49	1/472.9
Volume, V_{OR}	$d_{OR}^2 \cdot l_{OR}$	1/49	1/1310
Time scale	$l_{OR}^{1/2}$	1/1	1/1.664
Velocity	$l_{OR}^{1/2}$	1/1	1/1.664
Power/Volume	$l_{OR}^{-1/2}$	1/1	1.664
Heat flux	$l_{OR}^{-1/2}$	1/1	1.664
Core power	$a_{OR} \cdot l_{OR}^{1/2}$	1/49	1/787
Flow rate	$a_{OR} \cdot l_{OR}^{1/2}$	1/49	1/787
Pump head	l_{OR}	1/1	1/2.77
Pressure drop	l_{OR}	1/1	1/2.77

**Facility for Experimental Simulation
of Transients and Accidents

SMART-ITL (Integral Test Loop)

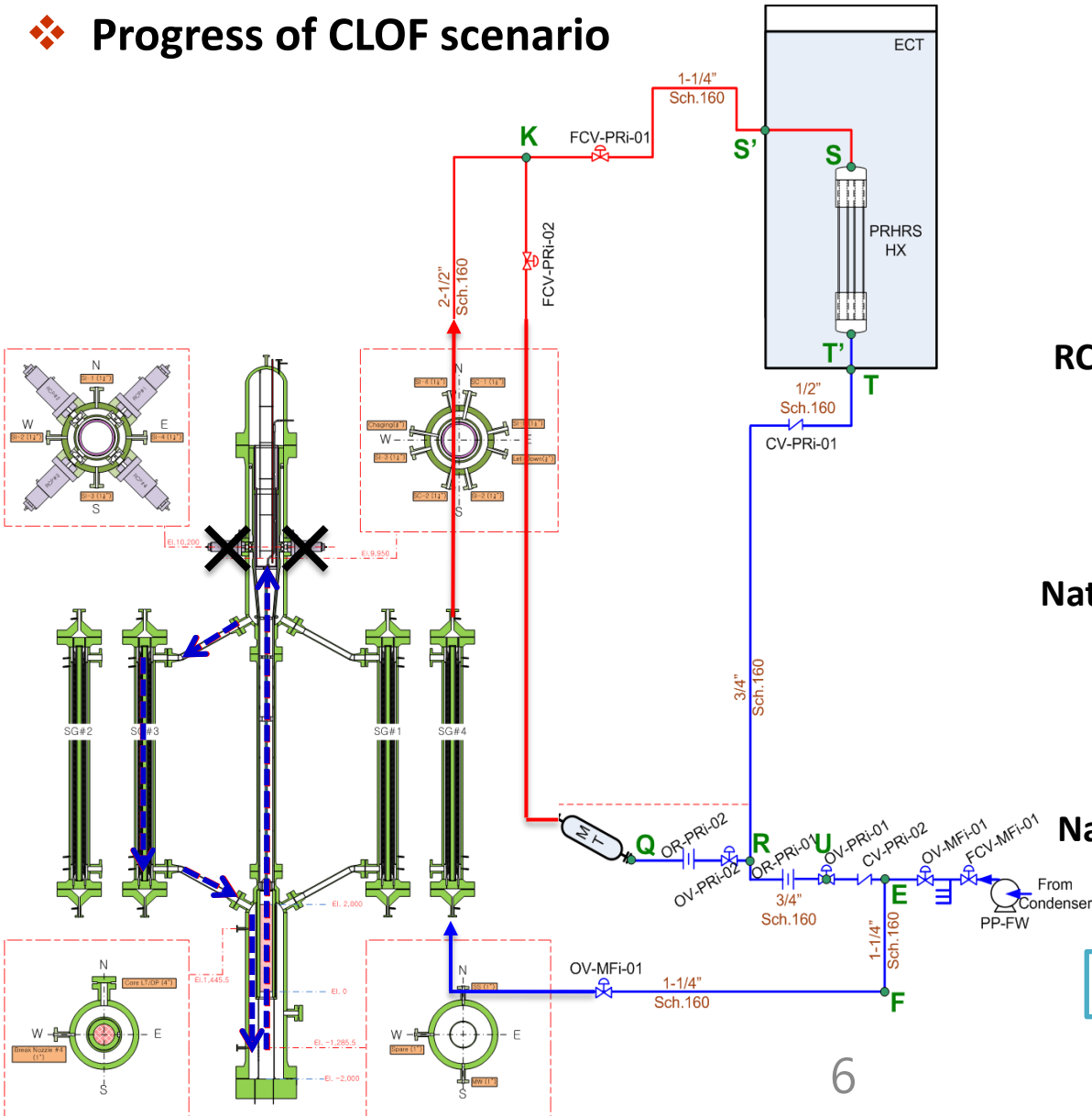
❖ Design Figures

- Design pressure & temp.:
 - 180 bar, 370 °C
- Core heater power:
 - Maximum: 2.0 MW (30% of scaled full power)
 - Operation: 1.5 MW (20%) + heat loss
- External SGs
 - Proper instr. and easy maintenance
- SG & PRHRS: 4 Trains
- PSIS (CMT & SIT): 4 Trains
- ADS: 2 Trains
- Major components
 - Reactor Coolant/Secondary systems
 - PRHRS, ASIS/PSIS
 - Auxiliary systems
 - Break system, Break meas. System, Break Pool
- Instruments : ~ 1,344
 - P, T, flow rates, mass, power, etc.



CLOF (Complete Loss Of RCS Flow rate)

Progress of CLOF scenario



Steady-state operation



RCPs stop due to emergency power loss.



Natural Circulation occurs in primary loop.



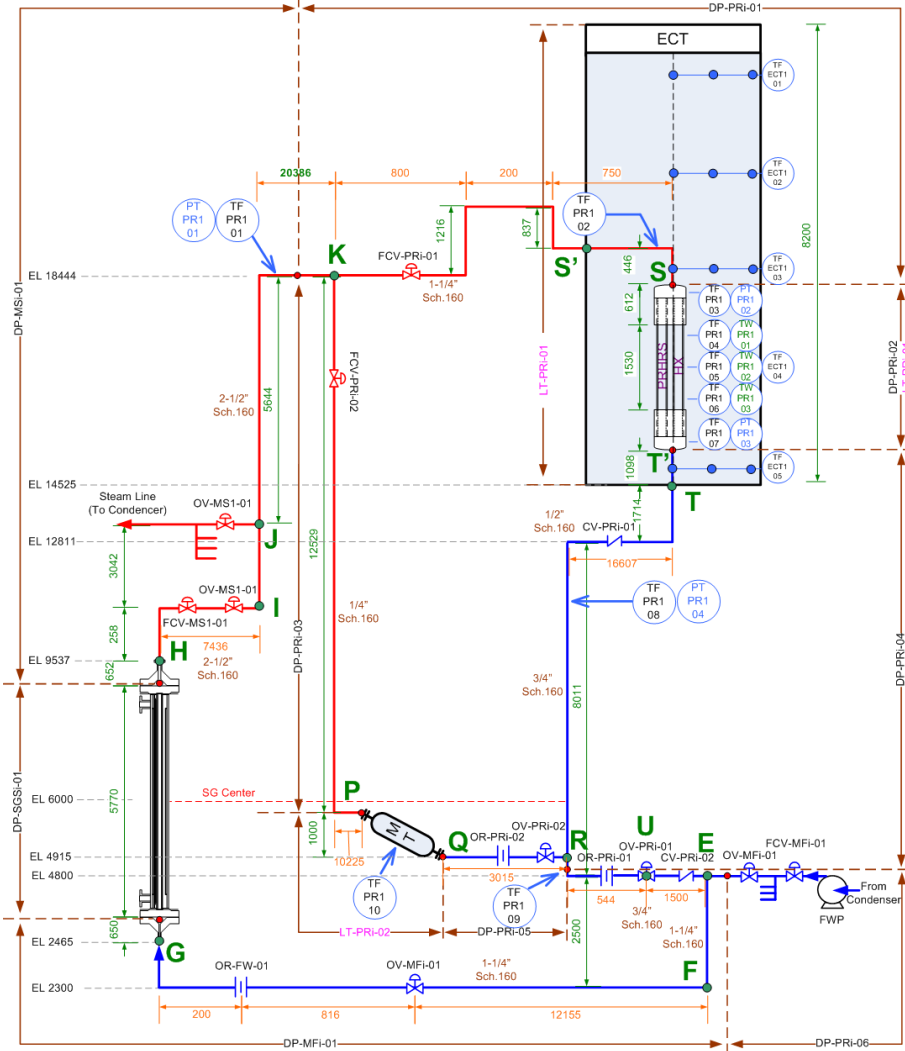
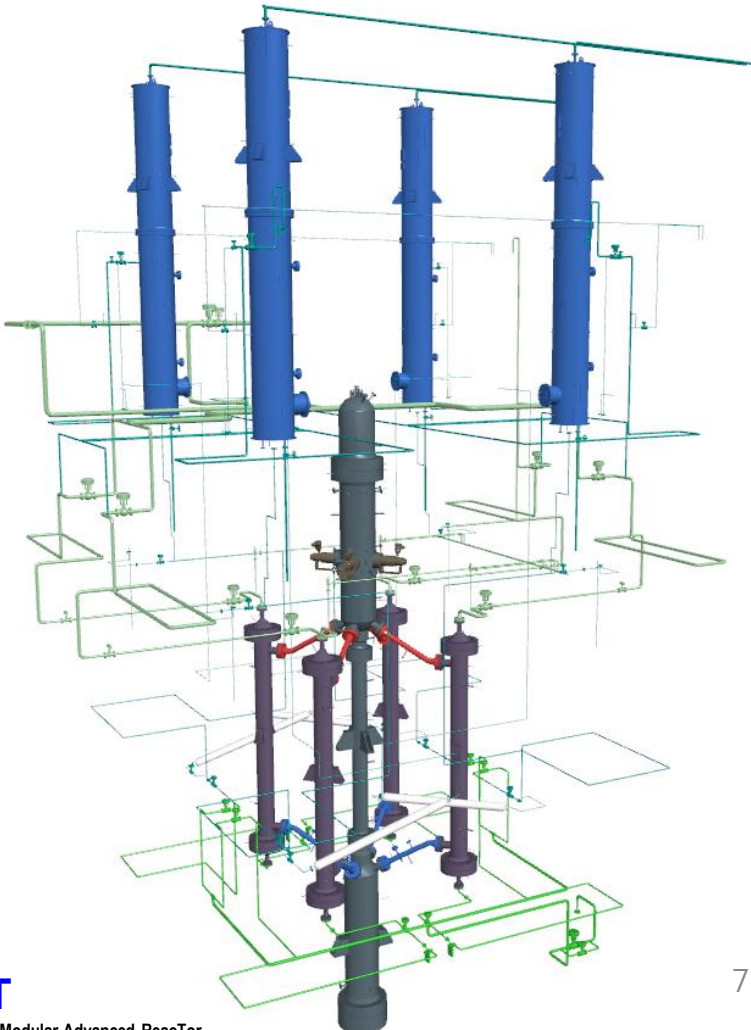
Natural Circulations in PRHRS also start.

Single phase NC vs. Two-phase NC

PRHRS of SMART-ITL (1/2)

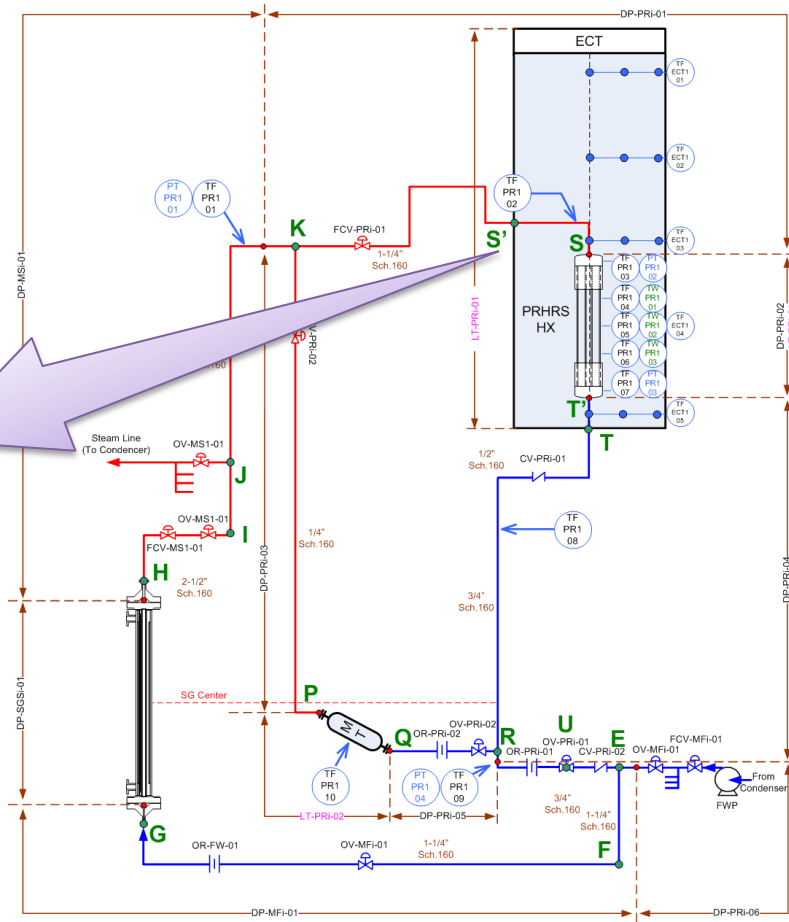
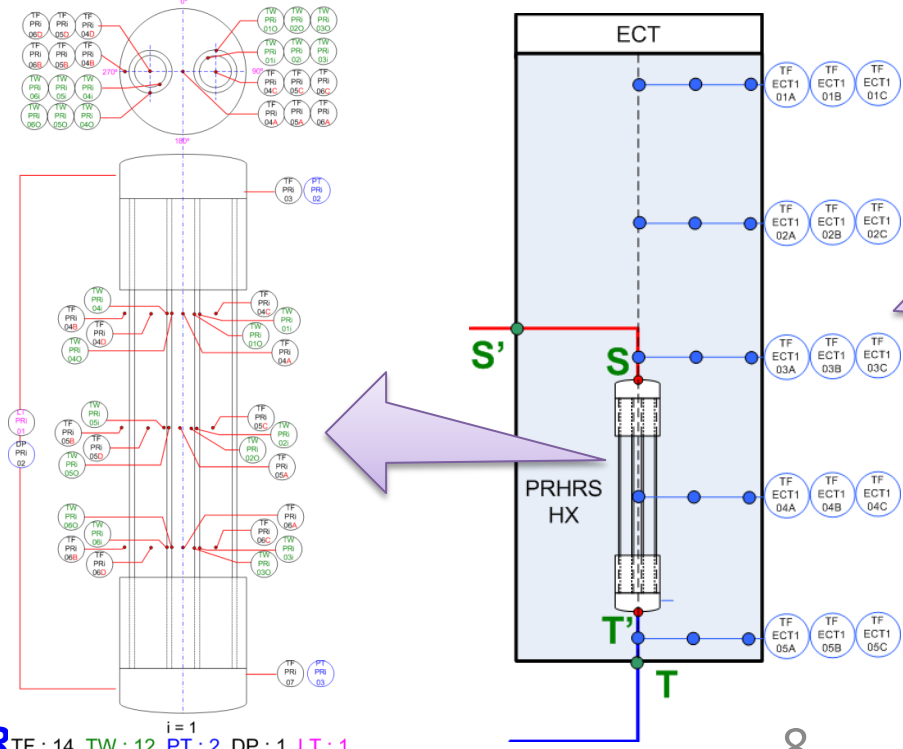
❖ Passive Residual Heat Removal System (PRHRS)

- Heat exchanger (Hx) + Emergency Cooldown Tank (ECT) + Makeup tank (MT): 4 trains



❖ Passive Residual Heat Removal System (PRHRS)

- Heat removal by two-phase natural circulation
 - < safety shutdown temperature (176 °C) in 36 hrs
 - Maintain < SST during 72 hrs without MT.
- Heat capacity of ECT is oversized for safety margin.



1. Introduction: SMATR-ITL & PRHRS

2. EXPERIMENT: CLOF TEST

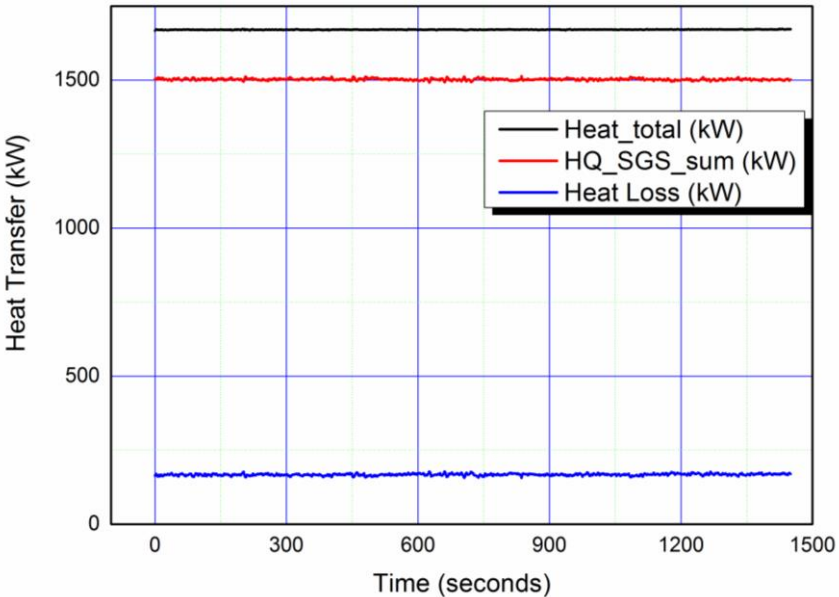


3. Results & Analysis

4. Conclusion & Further Study

Steady State Operation

Parameter	Target Value	Measured Value
Core Power (MW)	1.50	1.67 (Heat Loss 0.17)
Core Coolant Temperatures (In / Out) (°C)	295.5 / 320.9	295.5 / 320.6
S/G Coolant Temperatures (In / Out) (°C)	320.9 / 295.5	320.6 / 298.1
Mass Flow Rate of Primary Coolant, kg/s	10.23	10.26
Pressure of PZR, MPa	15.00	15.05
Coolant Temp. of PZR, (°C)	342.1	342.1
Volume of Coolant in PZR, % (m)	70 (3.12)	70.6 (3.14)



Heat Balance (Avg.)

Heat source from core	1670kW
Heat sink of SG	1502kW
Heat loss	168kW

Sequence of Event

Event	Trip signal and set-point	Operation
Transient occurs Ex.) Station black-out	RCP stop & RCP coast-down FW pump stop Turbine stop	PP-RC01 ~ 04 STOP PP-MF-01 STOP
RCP Pump Signal (RPS) set-point	RCP stop + 0.37 s	$RPM = 0.9 \times RPM_{normal}$
Reactor trip signal PRHRAS actuation signal (PRHRAS) CMT actuation signal (CMTAS) MSIV/FIV close start PRHRAS IV open start	RPS + 1.1 s	PRHRASAS, CMTAS due to low feed water flow rate
Control rod insert	RPS + 1.6 s	Decay heat (residual heat) table
4 trains of CMT injection start	RPS + 2.2 s (CMTAS + 1.1 s)	OV-IL1,2,3,4-101 OPEN
MSIV/FIV close completed PRHRAS IV open completed	RPS + 6.1 s (PRHRAS + 5.0 s)	OV-PR 1,2,3,4-03 OPEN OV-MS 1,2,3,4-01 CLOSE OV-MF 1,2,3,4-01 CLOSE
End of event	PRHRAS + 36 hr (Temp. of coolant < 215 °C)	After safety shut down condition

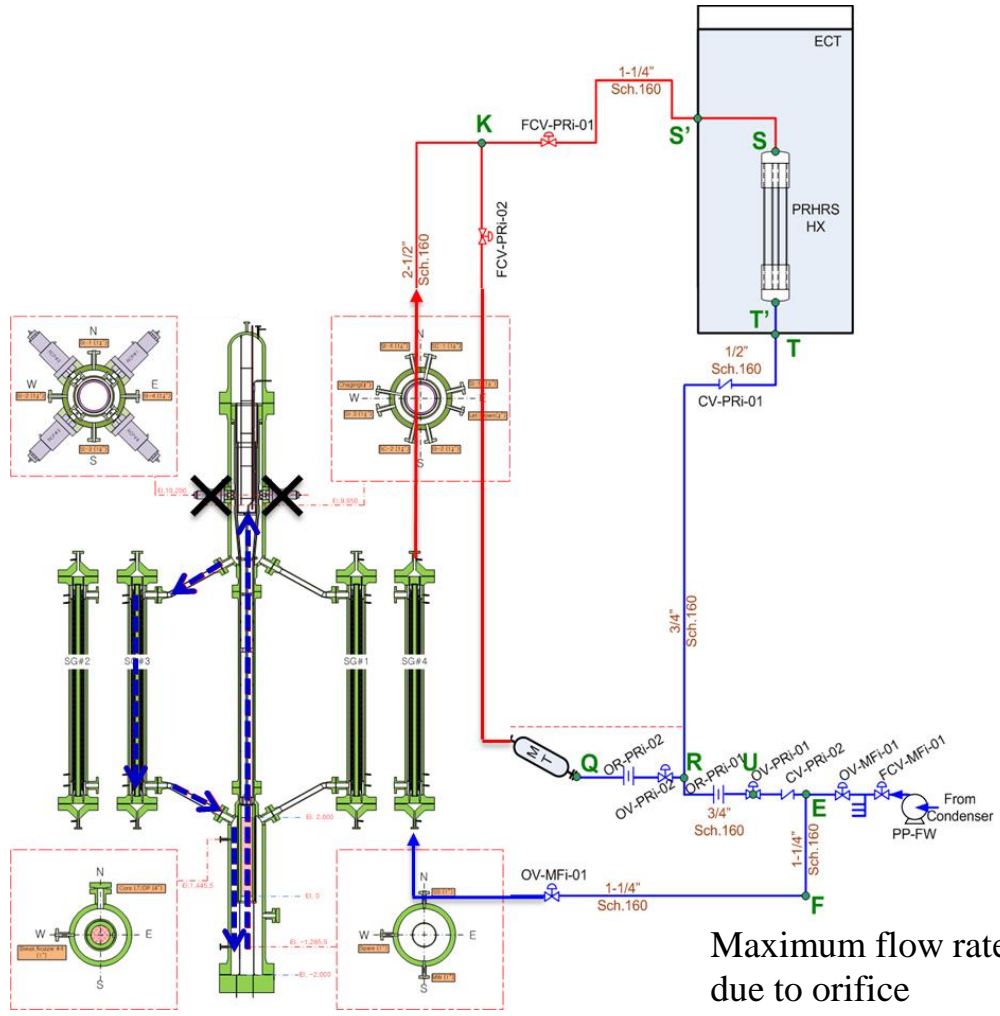
1. Introduction: SMATR-ITL & PRHRS
2. Experiment: CLOF Test

3. RESULT & ANALYSIS

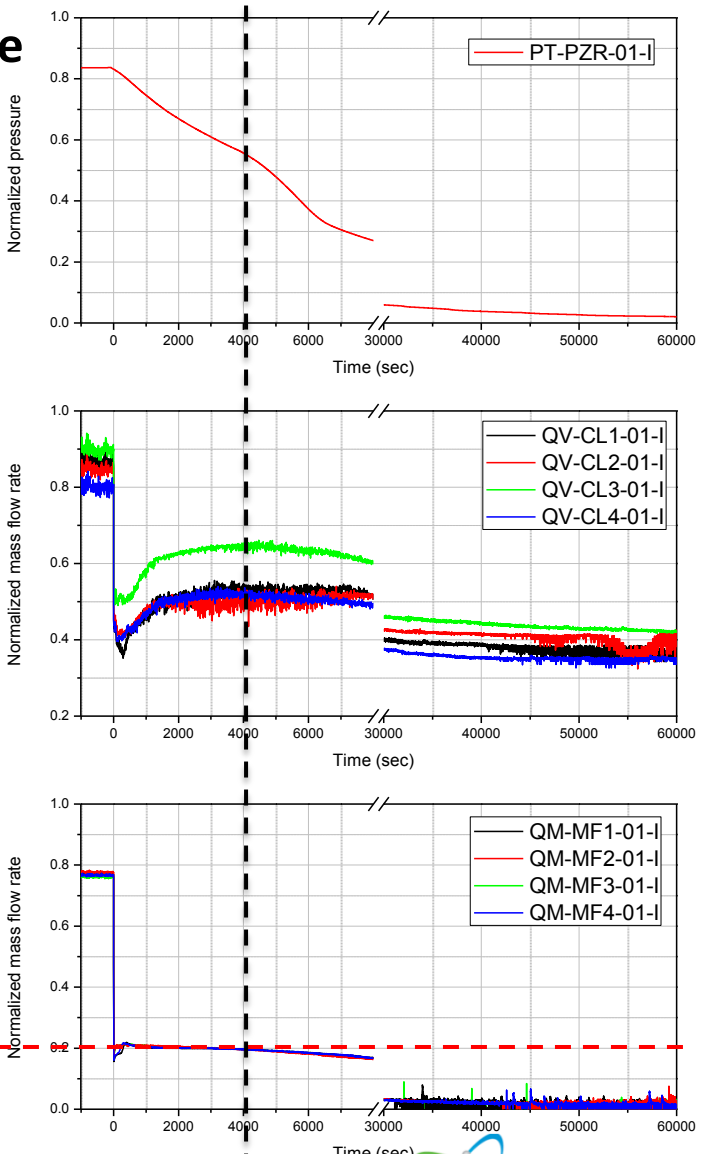


4. Conclusion & Further Study

Transient Simulation: Decay Power & Pressure



Maximum flow rate due to orifice



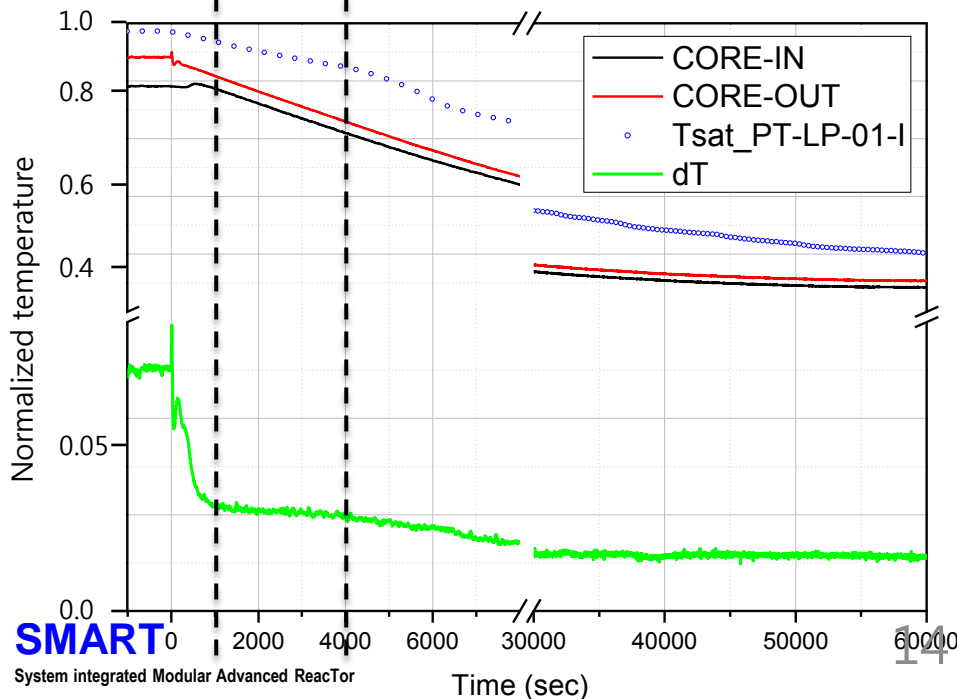
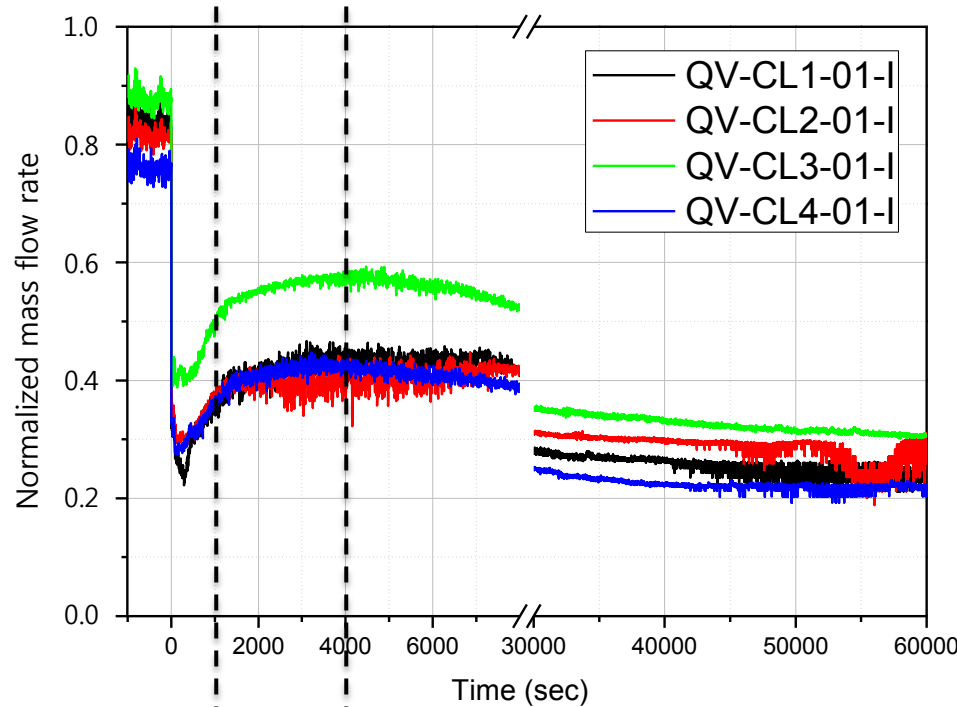
Results & Analysis

❖ Mass Flow Rate & Temp. (1ry)

$$Ra_L = Gr_L Pr = \frac{(\beta\Delta T)gL^3}{\nu^2} Pr$$

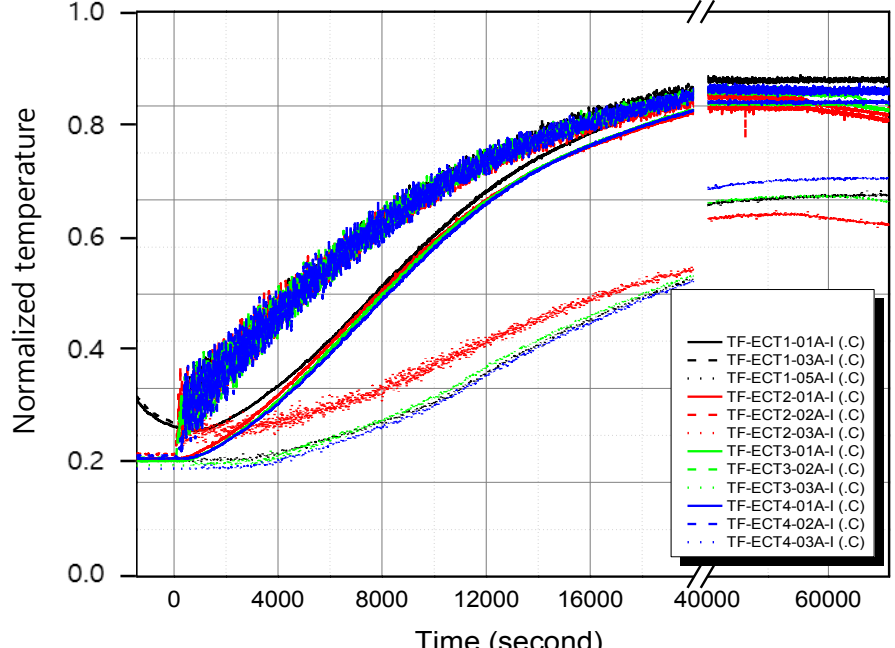
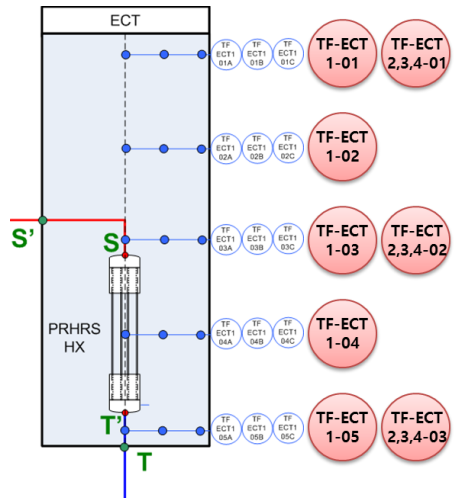
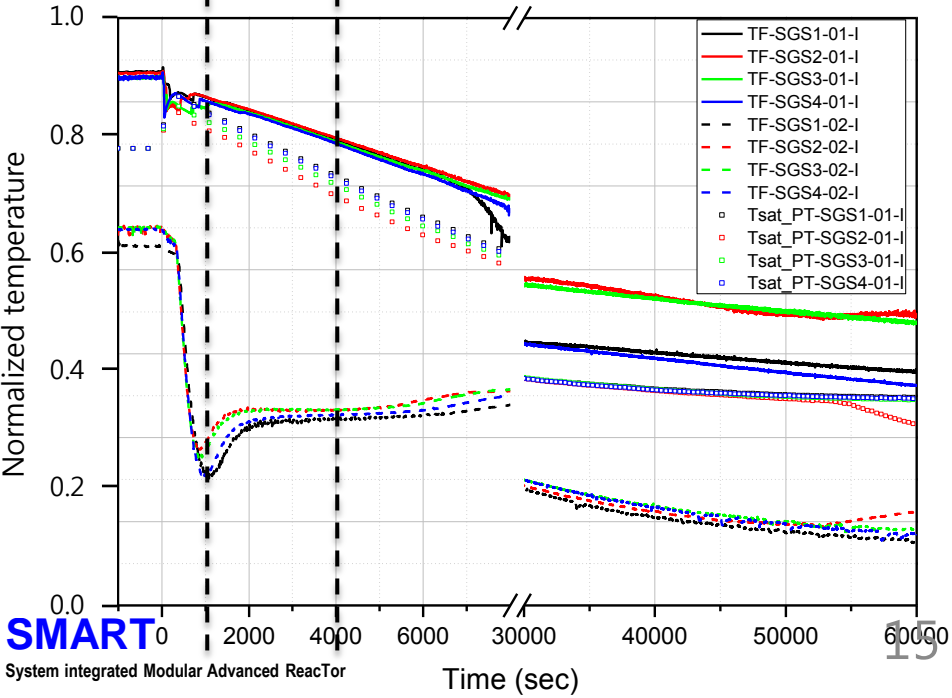
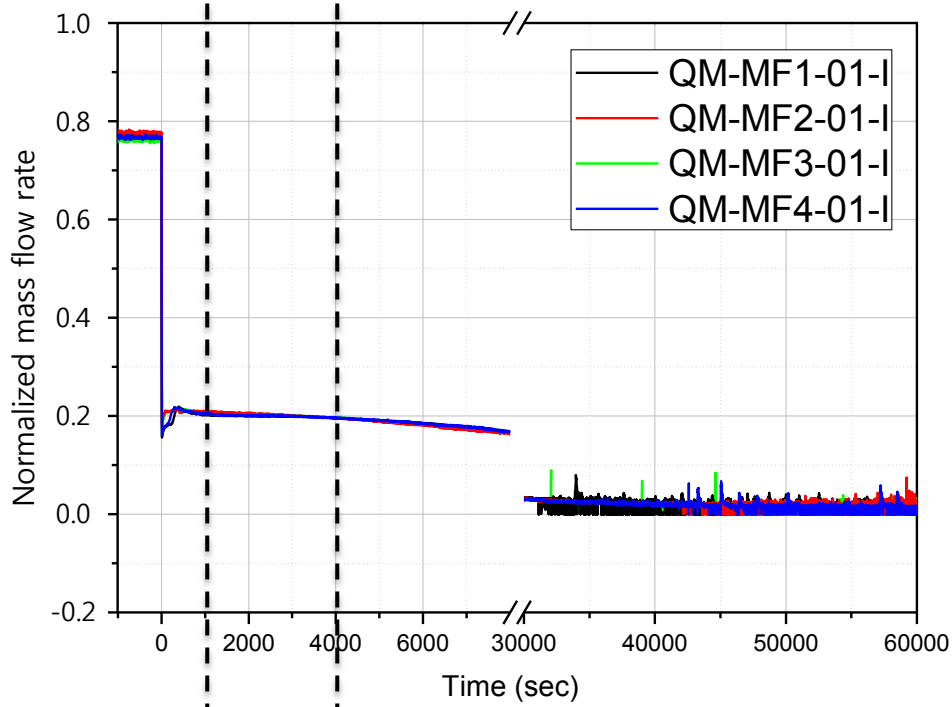
- < 4000 s : NC mass flow in primary increases.
- > 4000 s : NC mass flow in primary decreases.

➔ Temperature potential affects NC flow rate.

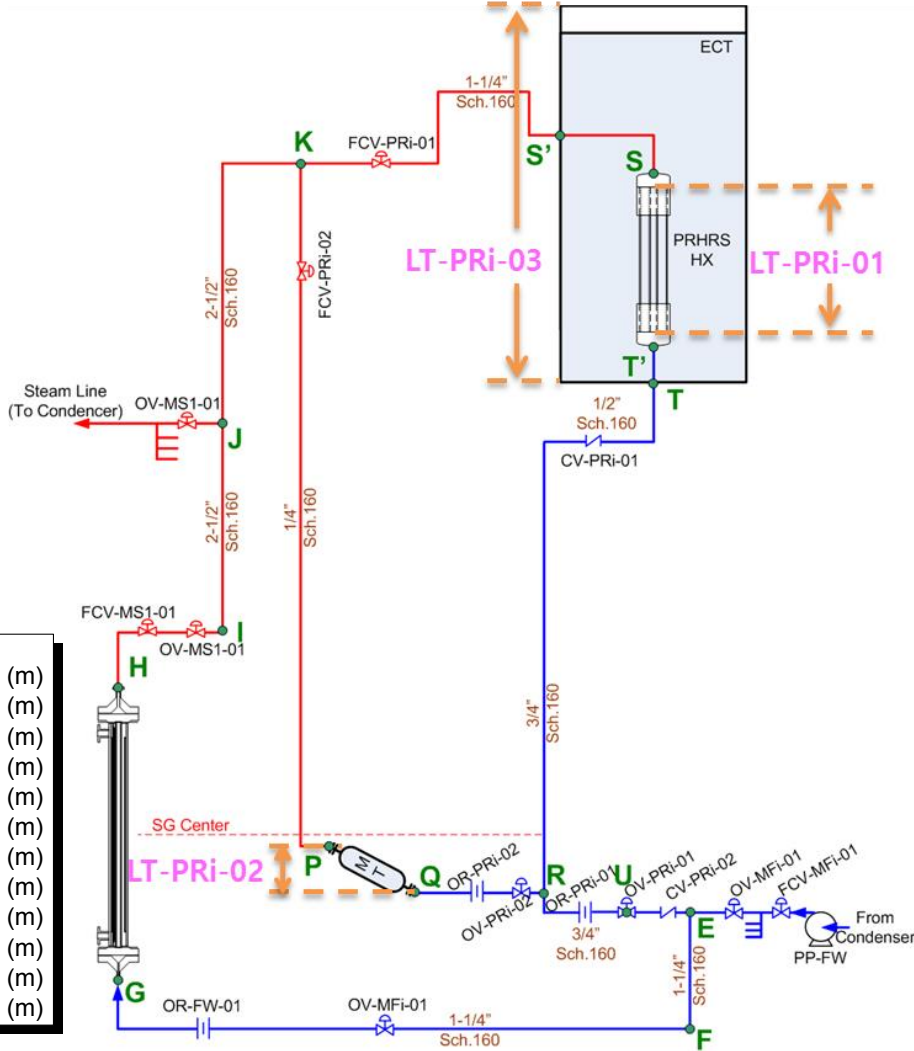
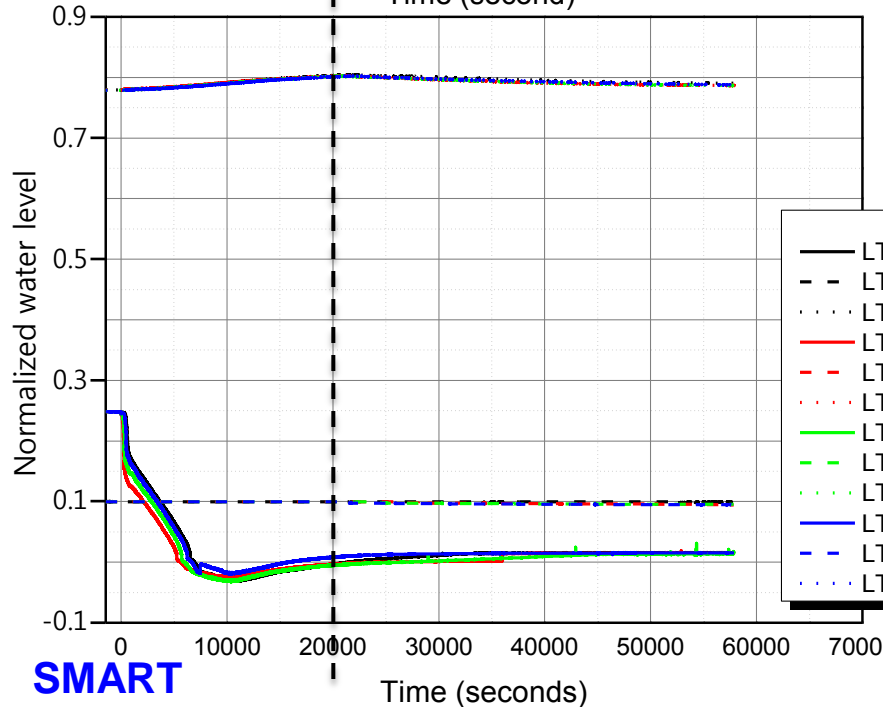
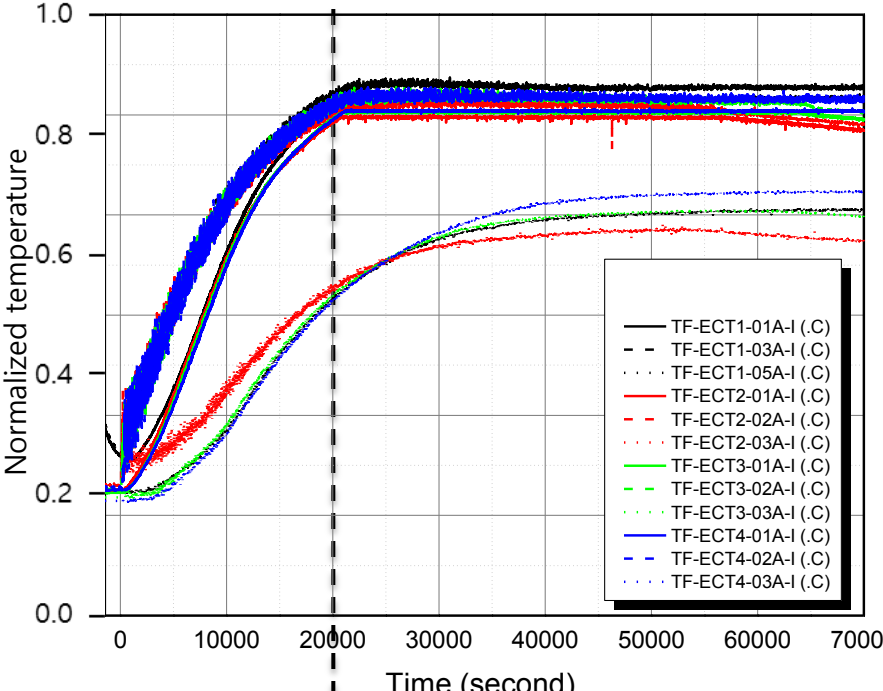


Results & Analysis

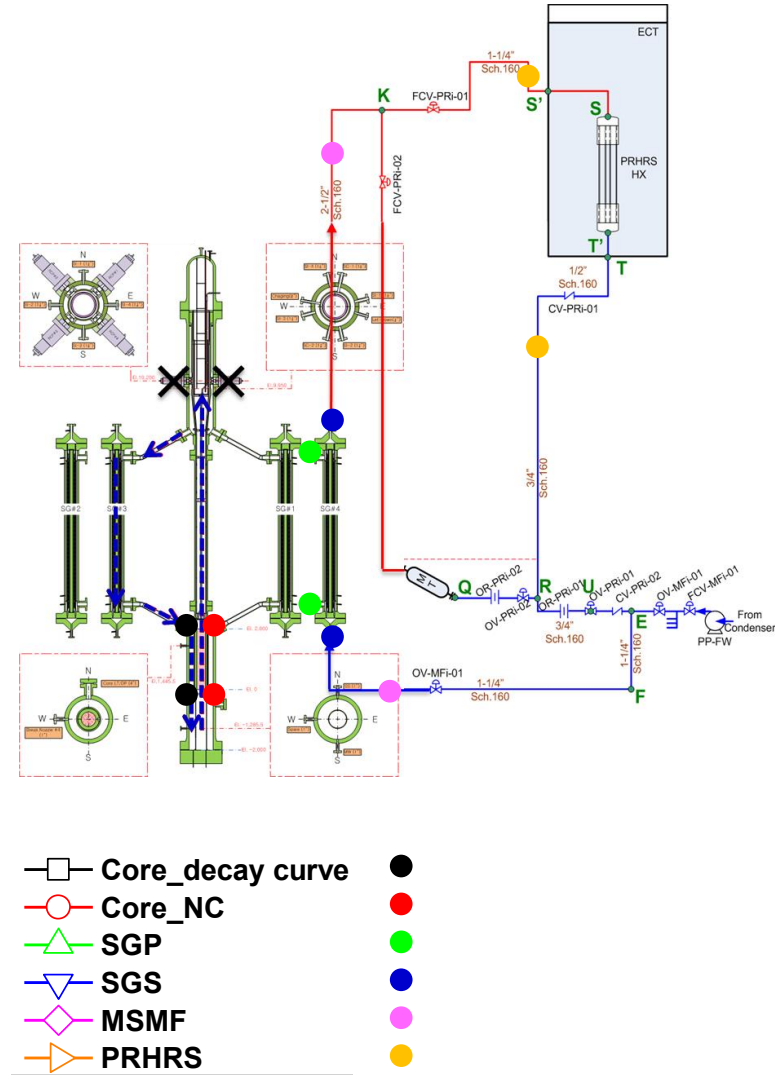
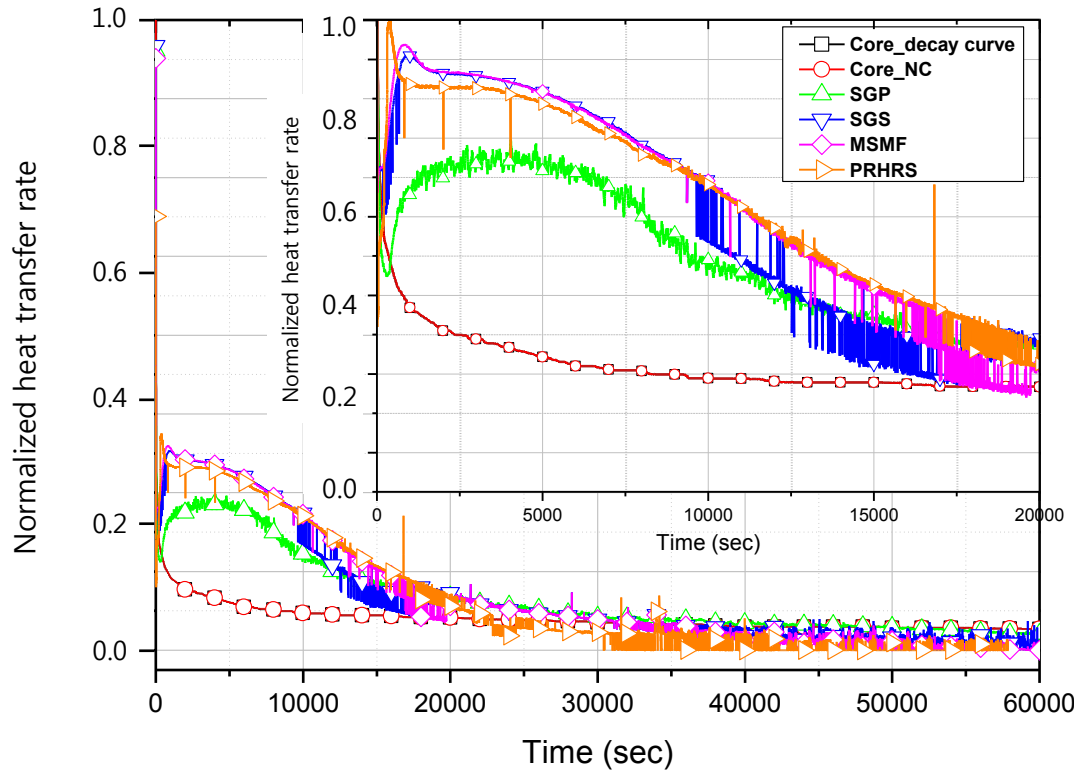
❖ Mass Flow Rate & Temp. (2ry)



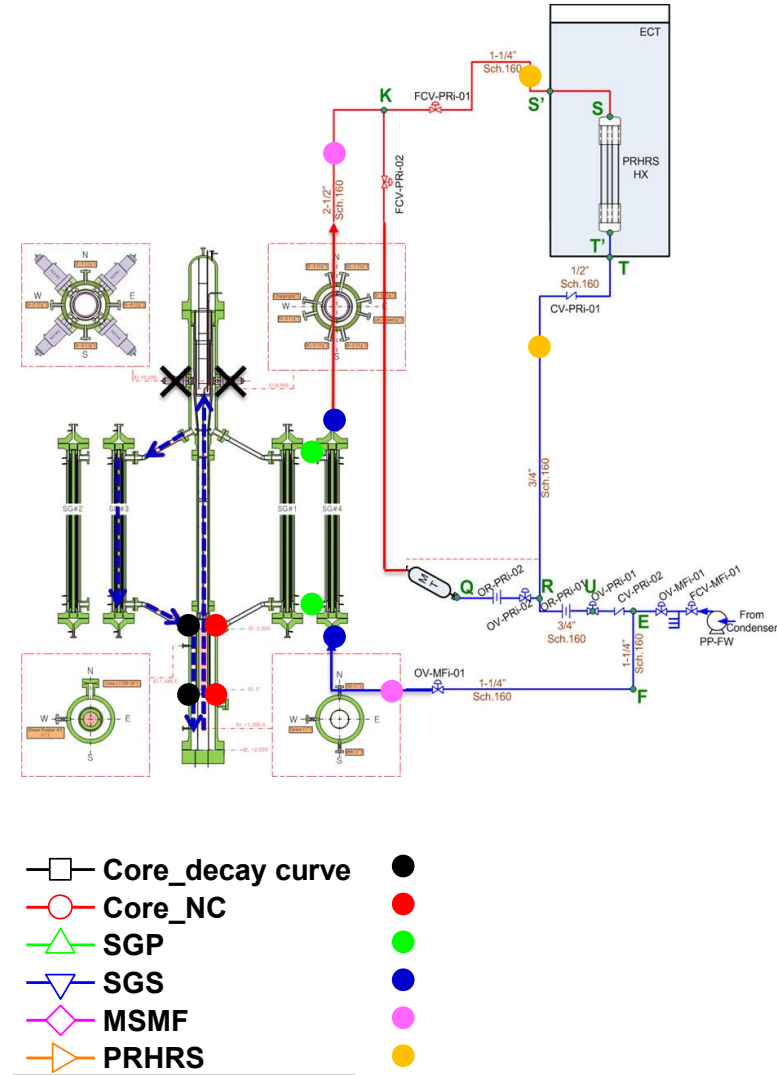
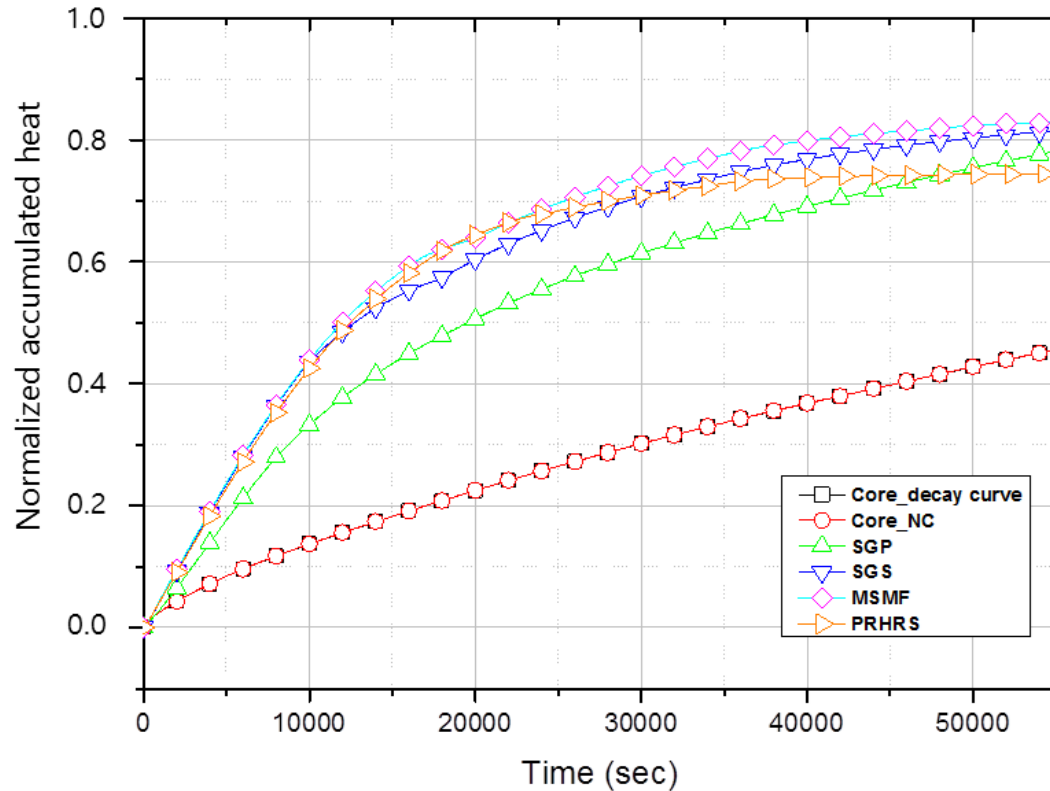
❖ Temp. & Water Level of PRHRs



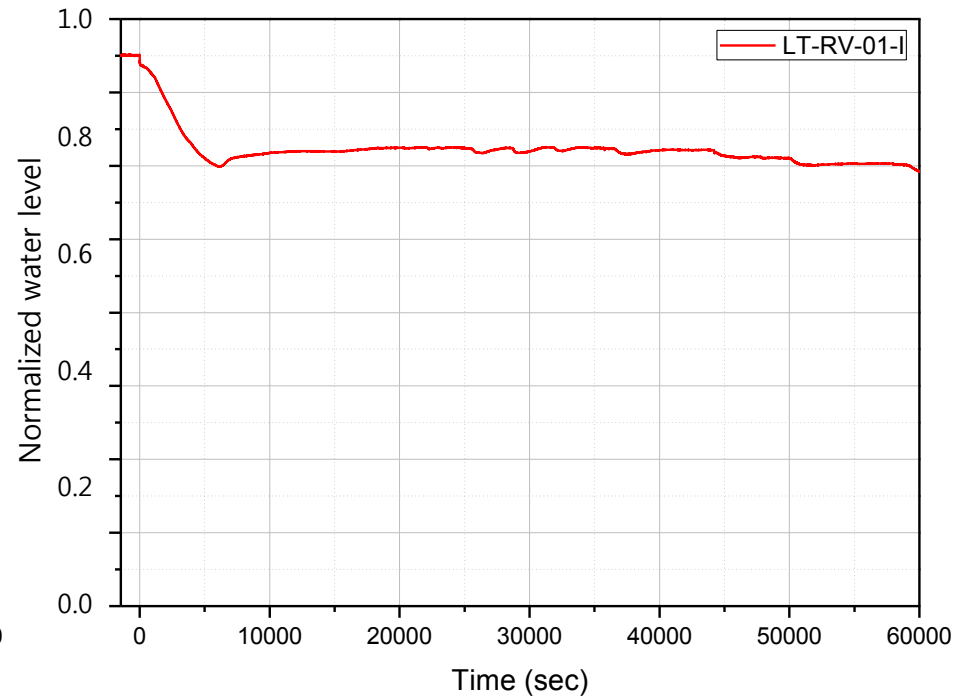
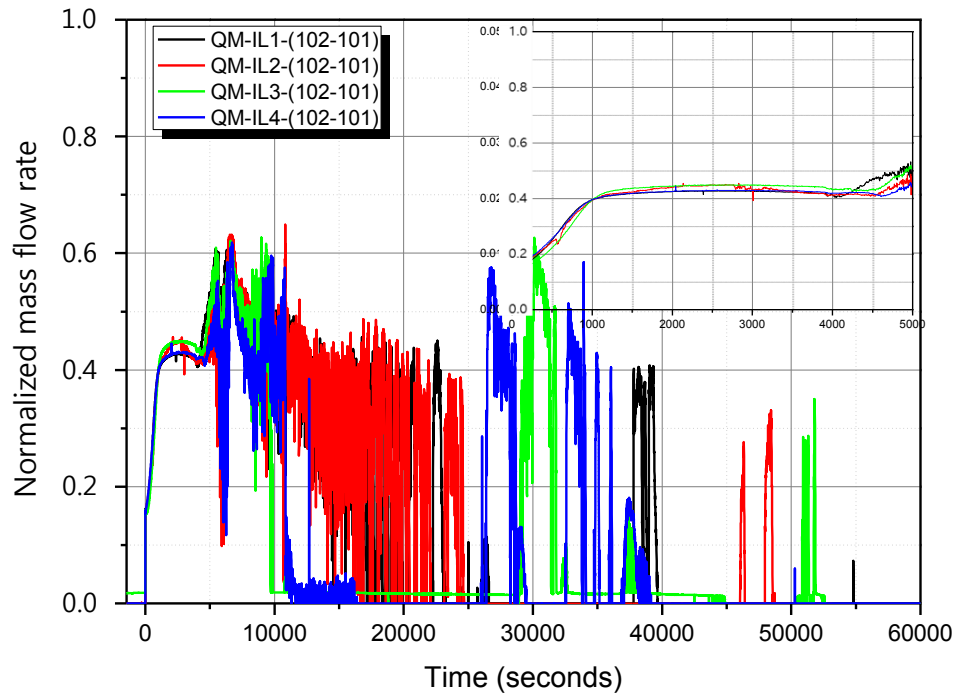
❖ Heat Balance between 1ry & 2ry systems



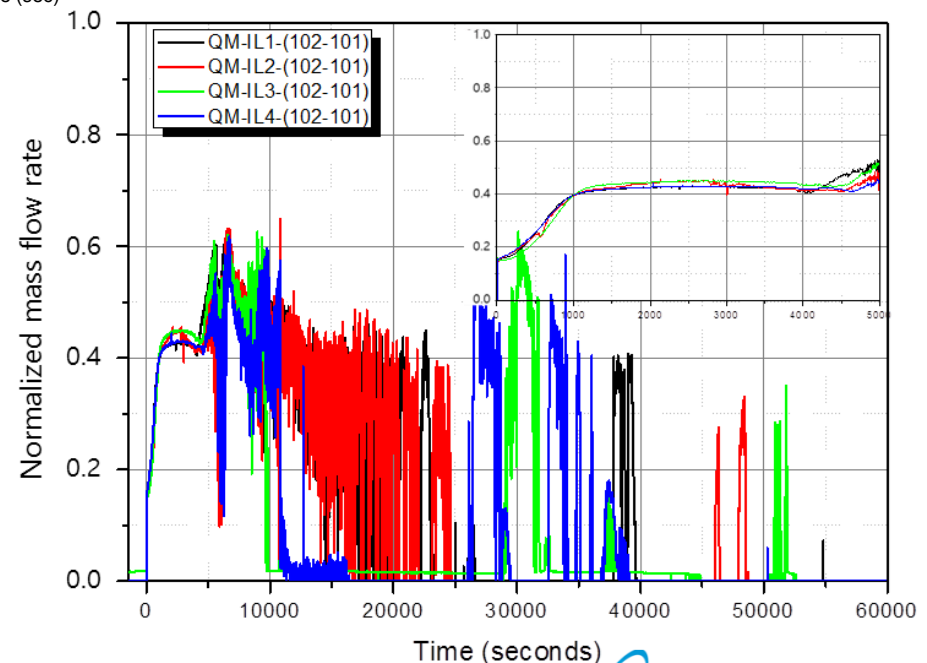
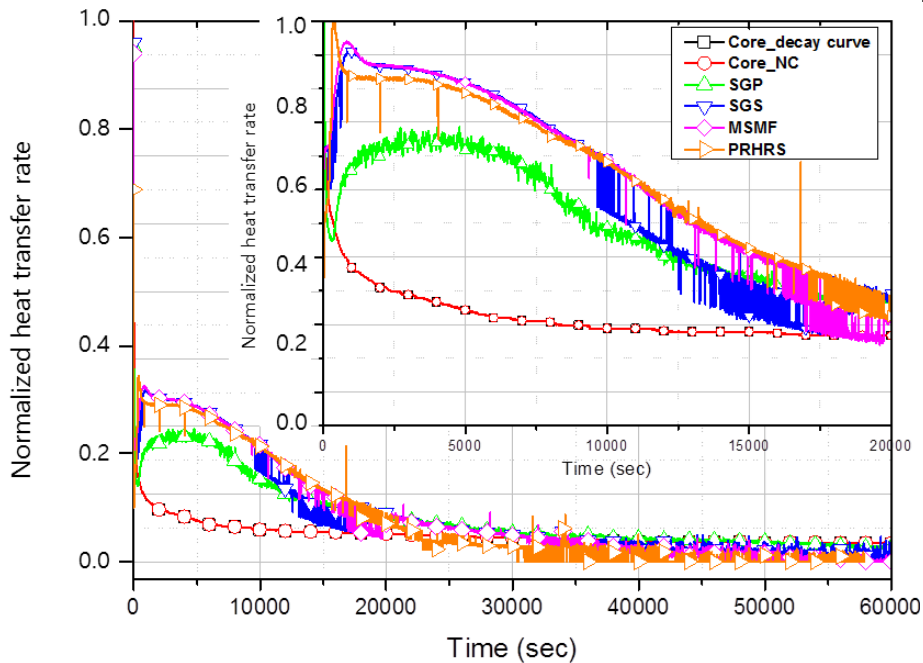
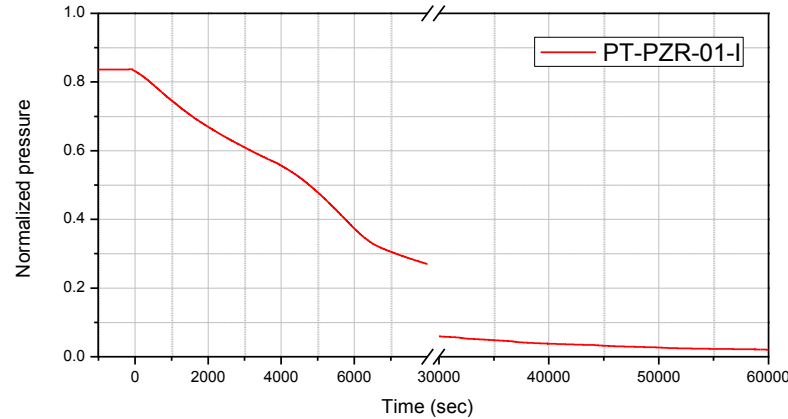
Heat Balance between 1ry & 2ry systems



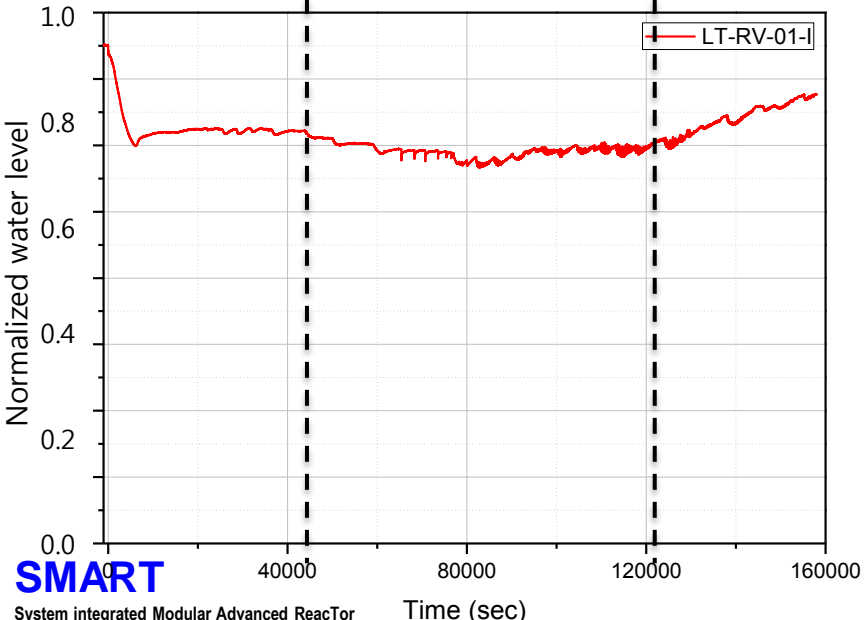
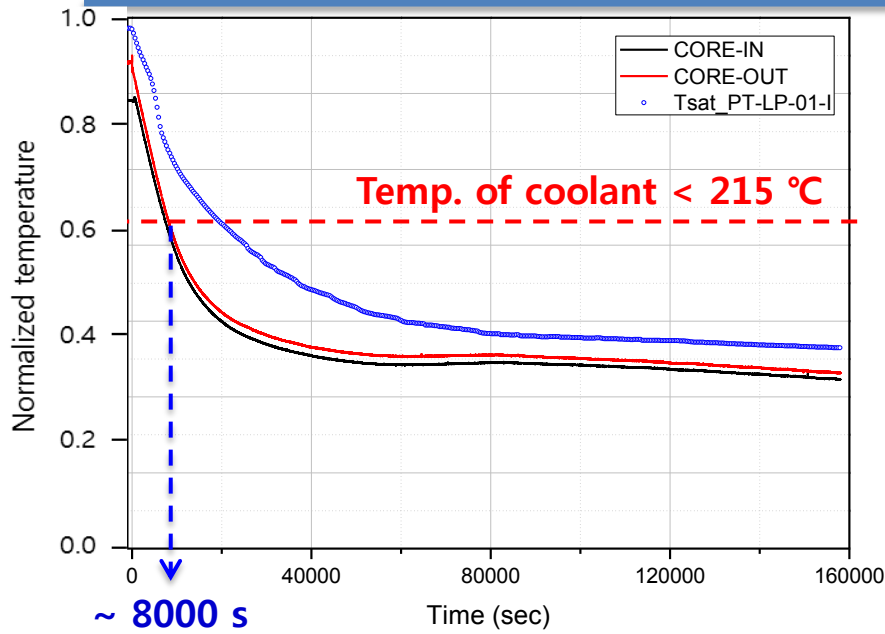
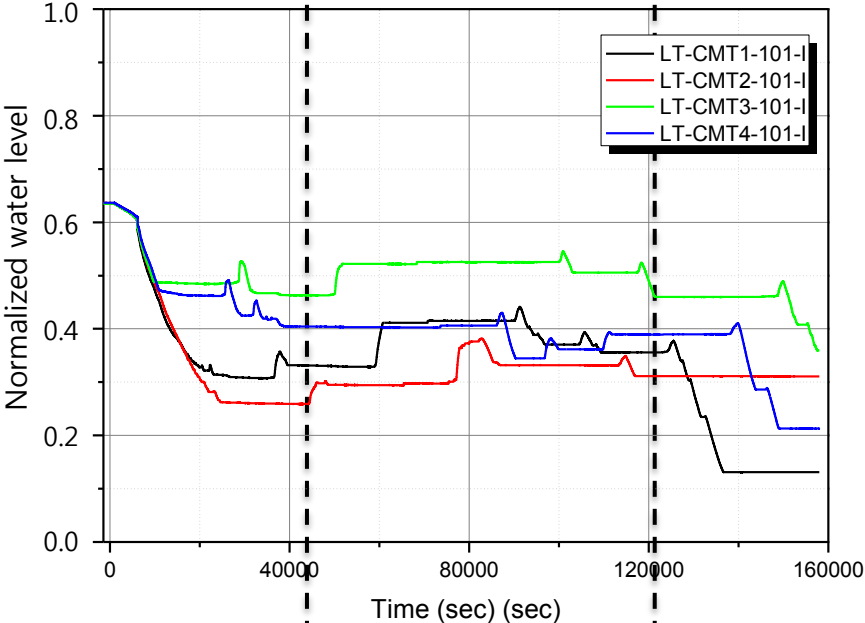
❖ Characteristic of CMT Injection



Physical Basis of Pressure Gradient Change



Results & Analysis



Safety criteria were satisfied!!

- 1) Core was not exposed during 36 hrs with PRHRS.
- 2) Temperature of primary coolant was sustained under safety shut down temperature (< 215 °C).

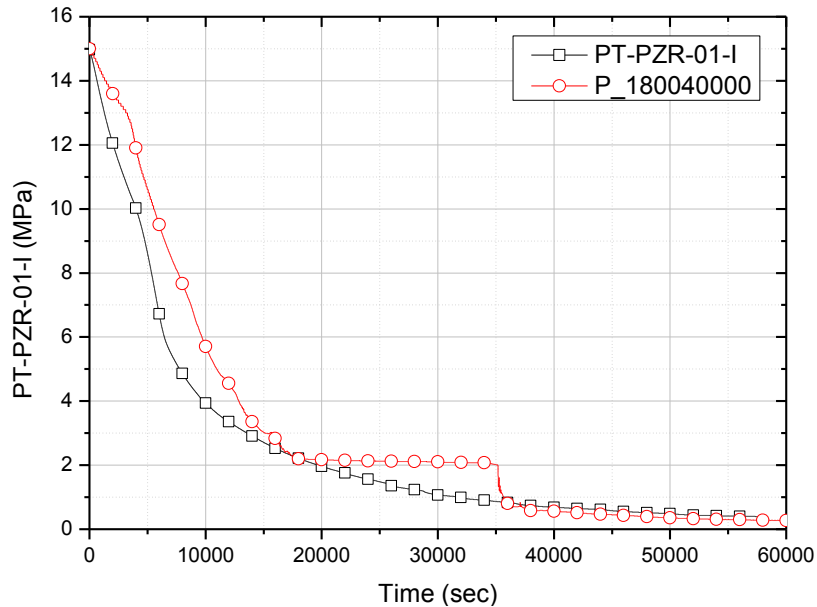
1. Introduction: SMATR-ITL & PRHRS
2. Experiment: CLOF Test
3. Results & Analysis

4. CONCLUSION & FURTHER STUDY



- ❖ CLOF accident was simulated with SMART-ITL.
- ❖ Natural circulations in the 1ry & 2ry system were analyzed with trend of pressure gradient.
- ❖ Heat balance from core (heat source) to ECT of PRHRS (heat sink) was quantified by experimental results and it was helpful to understand about progress of CLOF accident.
- ❖ Effect of CMT injection couldn't be quantified independently, but it was an important factor to remain the water level of RCS.
- ❖ If CLOF occurred in the SMART, the passive safety systems (4 trains of PRHRS & CMT) were enough to ensure safety of nuclear reactor.

❖ Simulation with system analysis code (MARS)



❖ Study about local phenomena

- Subcooled boiling after CLOF accident
- Heat transfer in the ECT

