

## Comparison of ENDF/B-VIII.0 and ENDF/B-VII.1 libraries with MCS code.

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### 1. Introduction

Recently, ENDF/B-VIII.0 library was released by Cross Section Evaluation Working Group (CSEWG). The ENDF library is used to calculate Monte Carlo codes. It is important to verify the cross-section library before performing Monte Carlo calculations.

This paper presents comparison of ENDF/B-VIII.0 library and the ENDF/B-VII.1 library using ICSBEP model, VERA benchmark, PMR model. This process is important for ensuring the reliability of the ENDF/B-VIII.0 library. MCS code was used to compare ENDF/B-VIII.0 library and ENDF/B-VII.1 library. Some of ICSBEP model, VERA benchmark model and PMR model were used for the calculation. The ENDF/B-VIII.0 library was generated using NJOY code.

### 2. Verification of ENDF/B-VIII.0 library

This section describes the newly released ENDF/B-VIII.0 and the MCS and NJOY codes used in this study. Also, the ICSBEP benchmark model used in calculation was described.

#### 2.1 ENDF/B-VIII.0 library

ENDF/B-VIII.0 was released in early 2018, incorporating work from the US national and international nuclear science communities. This library has been published in the form of the ENDF/B-VI.0 format and a new generalized nuclear database structure. Unlike ENDF/B-VII.1, ENDF/B-VIII.0 had significant changes in the neutron response to major actinides and other nuclides that affect the simulation of nuclear criticality. The isotope <sup>1</sup>H, <sup>16</sup>O, <sup>56</sup>Fe, <sup>235,238</sup>U, <sup>239</sup>Pu reflected results of ENDF/B-VIII.0 in Collaborative International Evaluation Library Organization (CIELO). Neutron standards and thermal scattering libraries are included in ENDF/B-VIII.0. In the Table I, the neutron sublibrary has expanded 32% to contain 557 evaluations (see Table I.) [1].

Table I. Number of Nuclides Provided Each ENDF/B Library in Each Sublibrary [1].

Sublibrary	VIII.0	VII.1	VII.0	VI.8
Neutron	557	423	393	328
Thermal n-scattering	33	21	20	15
Proton	49	48	48	35

Deuteron	5	5	5	2
Triton	5	3	3	1
Helium3	3	2	2	1
Alpha	1	n/a	n/a	n/a
Photonuclear	163	163	163	n/a

#### 2.2 MCS code

MCS code has been developed for the purpose of large scale reactor analysis with accelerated Monte Carlo simulation. MCS uses continuous energy cross-section libraries and detailed geometrical data to estimate neutronics design parameters of a nuclear reactor such as effective multiplication factor, neutron flux, and fission power [2].

#### 2.3 NJOY code

NJOY code is a comprehensive computer code package for calculating continuous energy cross-section, multi-group cross-section from evaluated nuclear data. NJOY code generates libraries for variety of particle transport and analysis codes, using files evaluated for neutrons, photons, and charged particles [3].

#### 2.4 Specification of the ICSBEP model

Some of the International Criticality Safety Benchmark Experimental Problem (ICSBEP) was selected for verification of each library. ICSBEP is one of the criticality benchmark. Figure 1 shows assembly of LMT007 case1, one of the various models used in the calculations. Figure 2 shows vertical cross-section view of rod. The specification of LMT007 case1 model are described in Table II.

Table II. Specification of LMT007 case1 model

Parameter	Value
Fuel rod diameter. (cm)	0.7729
Fuel rod length. (cm)	30
Fuel density. (g/cm <sup>3</sup> )	18.974
Bottom reflector. (cm)	21.59
Moderator density. (g/cm <sup>3</sup> )	0.997771
Pin pitch (cm)	1.30
Number of rods	394
Total height (cm)	60.49
Top reflector height (cm)	8.90

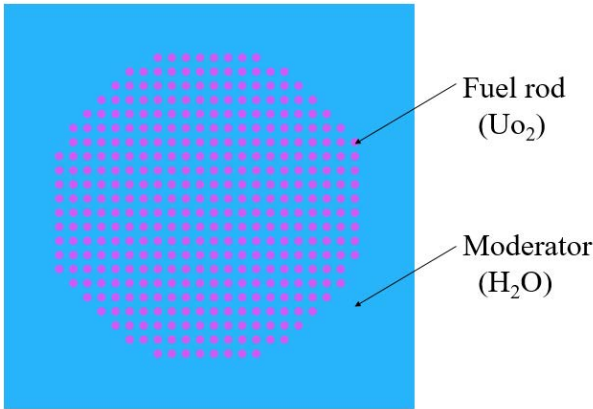


Fig 1. LMT007 case1 assembly design

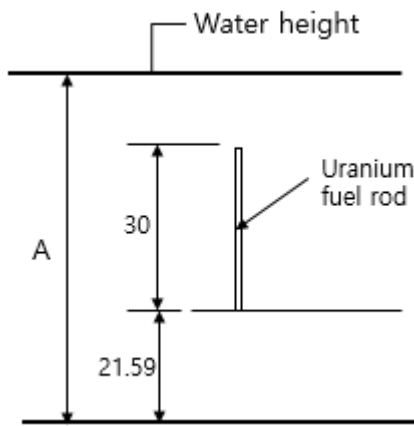


Fig 2. Vertical cross-sectional view of rod configuration with infinite radial water reflector. [4]

### 3. Numerical Results

Table III-X shows results of the ICSBEP model. The model used for calculation was selected from some of the ICSBEP model. The multiplication factor ( $k_{eff}$ ) values of Table III-X shows that ENDF/B-VIII.0 values is less than ENDF/B-VII.1 value substantially. The deviations of the  $k_{eff}$  values are about 20 pcm. Table III-X shows the result of comparing ENDF/B-VIII.0 and ENDF/B-VII.1 using MCS code. In Table III-X, 500 inactive cycles and 2000 active cycles were used with a neutron history of 10000 per cycle. The temperature was assumed to be 300K.

In order to compare ENDF/B-VIII.0 library and ENDF/B-VII.1 library, other models were selected. Selected models are VERA benchmark model and PMR model. A material of fuel and cladding in VERA benchmark is uranium and zirconium. Table XI shows  $k_{eff}$  value of 2D pin-cell calculation. The deviations of the  $k_{eff}$  values are about 20 pcm. In Table XI, 20 inactive cycles and 100 active cycles were used with a neutron history of 50000 per cycle. The temperature was assumed to be 600K. Unlike the other models, the fuel type of PMR model is TRISO particle. Table XII shows  $k_{eff}$  value of PMR model. The deviations of the  $k_{eff}$  values are about 26 pcm. In Table XII, 20 inactive

cycles and 100 active cycles were used with a neutron history of 50000 per cycle. The temperature was assumed to be 900K.

Table III.  $k_{eff}$  Results in LCT 001 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	1.00030	1.00075	-45
Case02	0.99972	1.00019	-47
Case03	0.99903	0.99971	-60
Case04	0.99944	1.00004	-60
Case05	0.99764	0.99831	-67
Case06	0.99960	1.00009	-49
Case07	0.99906	0.99938	-32
Case08	0.99790	0.99838	-48

Table IV.  $k_{eff}$  Results in LCT 008 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	1.00124	1.00245	-121
Case02	1.00182	1.00273	-91
Case03	1.00234	1.00300	-66
Case04	1.00141	1.00212	-71
Case05	1.00122	1.00200	-78
Case06	1.00117	1.00223	-106
Case07	1.00072	1.00141	-69
Case08	1.00018	1.00084	-66
Case09	1.00004	1.00141	-137
Case10	1.00119	1.00229	-110
Case11	1.00224	1.00286	-62
Case12	1.00140	1.00224	-84
Case13	1.00140	1.00224	-84
Case14	1.00192	1.00225	-33
Case15	1.00125	1.00206	-81
Case16	1.00149	1.00241	-91
Case17	1.00032	1.00124	-92

Table V.  $k_{eff}$  Results in LCT 017 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	1.00194	1.00267	-73
Case02	1.00158	1.00258	-100
Case03	1.00058	1.00114	-56
Case04	0.99855	0.99960	-105
Case05	0.99987	1.00098	-111
Case06	1.00041	1.00065	-24
Case07	1.00040	1.00125	-85
Case08	0.99922	0.99963	-41
Case09	0.99840	0.99878	-38
Case10	0.99878	0.99912	-34
Case11	0.99899	0.99937	-38
Case12	0.99891	0.99979	-88
Case13	0.99951	0.99995	-44

Case14	0.99981	1.00056	-75
Case15	0.99796	0.99818	-22
Case16	0.99906	0.99942	-36
Case17	1.00031	1.00117	-86
Case18	0.99900	0.99996	-96
Case19	0.99958	1.00016	-58
Case20	0.99858	0.99916	-58
Case21	0.99849	0.99967	-118
Case22	0.99781	0.99882	-101
Case23	1.00012	1.00095	-83
Case24	1.00099	1.00131	-32
Case25	0.99893	0.99972	-79
Case26	0.99637	0.99744	-107
Case27	0.99807	0.99969	-162
Case28	0.99966	0.99998	-32
Case29	0.99959	1.00114	-155

Table VI.  $k_{eff}$  Results in LCT 039 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	0.99701	0.99825	-124
Case02	0.99619	0.99707	-88
Case03	0.99518	0.99680	-162
Case04	0.99548	0.99700	-152
Case05	0.99697	0.99758	-61
Case06	0.99490	0.99560	-70
Case07	0.99503	0.99607	-104
Case08	0.99491	0.99609	-118
Case09	0.99441	0.99528	-87
Case10	0.99437	0.99545	-108
Case11	0.99464	0.99576	-112
Case12	0.99438	0.99555	-117
Case13	0.99408	0.99563	-155
Case14	0.99467	0.99586	-119
Case15	0.99488	0.99586	-98
Case16	0.99515	0.99657	-142
Case17	0.99467	0.99577	-110

Table VII.  $k_{eff}$  Results in LCT 050 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	0.99921	1.00078	-157
Case02	0.99915	1.00027	-112
Case03	0.99977	1.00095	-118
Case04	0.99935	1.00065	-130
Case05	1.00065	1.00191	-126
Case06	1.00045	1.00214	-169
Case07	1.00542	1.00655	-113
Case08	0.99776	0.99904	-128

Table VIII.  $k_{eff}$  Results in LCT 054 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	1.00247	1.00346	99
Case02	1.00250	1.00295	45
Case03	1.00239	1.00314	75
Case04	1.00253	1.00356	103
Case05	1.00202	1.00310	108
Case06	1.00208	1.00269	61
Case07	1.00228	1.00268	40
Case08	1.00189	1.00258	69

Table IX.  $k_{eff}$  Results in LCT 065 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case 02	1.00329	1.00508	-179
Case03	1.00242	1.00434	-192
Case04	1.00240	1.00496	-256
Case05	1.00327	1.00499	-172
Case06	1.00376	1.00505	-129
Case07	0.97706	0.97796	-90
Case08	1.00372	1.00486	-114
Case09	1.00281	1.00455	-174
Case10	1.00245	1.00368	-123
Case11	1.00273	1.00354	-81
Case12	1.00237	1.00369	-132
Case13	1.00199	1.00295	-96
Case14	1.00335	1.00430	-95
Case15	1.00300	1.00382	-82
Case16	1.00256	1.00381	-125
Case17	1.00227	1.00365	-138

Table X.  $k_{eff}$  Results in LCT 090 Model

Model name	ENDF/B-VIII.0 (A)	ENDF/B-VII.1 (B)	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
Case01	1.00251	1.00280	-29
Case02	1.00225	1.00305	-80
Case03	1.00239	1.00273	-34
Case04	1.00297	1.00373	-76
Case05	1.00245	1.00368	-123
Case06	1.00263	1.00323	-60
Case07	1.00193	1.00306	-113
Case08	1.00211	1.00315	-104
Case09	1.00243	1.00288	-45

Table XI. VERA Benchmark Model  $k_{eff}$

Model name	ENDF/B-VIII.0	ENDF/B-VII.1	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
VERA Benchmark	1.18380	1.18307	-73

Table XII. PMR Model  $k_{eff}$

Model name	ENDF/B-VIII.0	ENDF/B-VII.1	Diff (A)-(B) (pcm)
	$k_{eff}$	$k_{eff}$	
PMR Compact	1.25005	1.24817	118

#### 4. Conclusions

This paper presents comparison of ENDF/B-VIII.0 library and the ENDF/B-VII.1 library using ICSBEP model, VERA benchmark, PMR model.

Numerically, in the ICSBEP and VERA benchmark models, ENDF/B-VIII.0 showed lower  $k_{eff}$  value than ENDF/B-VII.1. The difference of two results is within statistical error. However, the results of PMR model showed that  $k_{eff}$  value was higher ENDF/B-VIII.0 than ENDF/B-VII.1.

#### REFERENCES

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