Assessment of TRACE VESSEL component for DVI direct bypass

Min Ki CHO (mkcho@kins.re.kr), Andong SHIN, Sara KANG

Korea Institute of Nuclear Safety, 62 Gwahak-ro, Yuseong-gu, Daejeon 34142, Republic of Korea

I. Introduction

- Reproducing the phenomena of Direct ECC bypass phenomenon is important for TH codes as the ECC bypass phenomenon is the one of the main safety issue for the Direct Vessel Injection system.
- Experimental investigations for the direct ECC bypass phenomenon [1,2,3,4]



III. Assessment results

A. VESSEL nodalization



II. Reference test and assessment methods

• MIDAS test

The MIDAS test facility is steam-water separate effect test facility could be simplified to five components; (1) boiler (steam supply system), (2) safety injection simulator (two activated SI lines; broken leg (#2) side & intact leg side (#4)), (3) test section (scaled down downcomer, cold-legs), (4) separator & containment simulator (sump for discharge flow from broken cold-leg of test section), (5) core barrel (sump for flow to lower part from outlet of test section). The maximum allowable operating conditions are 10 bars and 300 Celsius degrees

• Test modes

(1) Injection by DVI#2 and DVI #4 – 9 cases, (2) Injection by DVI#2 – 1 case, (3)) Injection by DVI#4 – 5 cases.

Abridgement of the MIDAS test results



B. Optional interfacial friction model



C. CCFL model

The MIDAS facility could be simplified with VESSEL, PIPE, FILL and BREAK components without heat structure as figure 3. The DVI lines and steam injection lines, intact cold-legs, are modeled with FILL and PIPE components. The broken cold-leg and the water sump are modeled with PIPE and BREAK components. The test section is modeled by VESSEL component with 15 axial cells and 2 radial cells. All faces of the first radial cells are blocked for representing downcomer shape. The mass flow rate and temperature of FILL components are assigned according to each test conditions.

The direct ECC bypass phenomenon is consequence of energy and momentum interaction of steam and water. The interfacial heat transfer and the interfacial friction should be assessed. However, there is no specific interfacial heat transfer model for VESSEL component except option for sensitivity analysis purpose. The effect of interfacial heat transfer would be assessed indirectly with other options. First of all, the effect of azimuthal VESSEL nodalization is considered. Modeling with 4channel, 6-channel and 12-channel were investigated. Two optional interfacial friction model for VESSEL component are investigated also which are IBLAUS option and LBDRAG option. Finally, effect of adopting CCFL models, the Kutateladze model and Wallis model, is investigated. [6,7]



Summary of calculation results

	Sensitivity factor	Base case	VESSEL node		Iterfacial friction		CCFL	
	Model	6-ch (bypass/cond.)	4-ch (bypass/cond.)	12-ch (bypass/cond.)	LBDRAG (bypass/cond.)	IBLAUS (bypass/cond.)	Wallis (bypass/cond.)	Kutateladze (bypass/cond.)
	Average of error(%)	46/22	35/24	63/19	46/22	28/22	92/20	45/22
	Average of STDEV	9.46E-4 /5.42E-3	1.38E-6 /4.50E-6	3.29E-2 /7.45E-3	6.67E-4 /1.51E-3	5.32E-1 /2.84E-2	7.01E-2 /4.97E-3	2.28E-2 /2.72E-3



Sensitivity factor





LBDRAG **IBLAUS**

CCFL model with 6-channel C.

Basic

model



IV. Conclusions

The assessment for the VESSEL component in TRACE V5 Patch5 is conducted using MIDAS test data. Effects of nodalization method and interfacial friction models and CCFL models are investigated in this study. The calculation result with 4 azimuthal channel showed the most stable and moderate data. Some optional models for interfacial friction or CCFL showed better accuracy than basic model in an average manner, however those results showed rather unstable quality. The basic models are recommended to simulate RPV with DVI while using VESSEL component.

V. REFERENCES

[1] P. Weiss et al. UPTF experiment refined PWR LOCA thermal-hydraulic scenarios: conclusions from a full-scale experimental program, Nuclear Engineering and Design, Vol. 149, p.335-347, 1994. [2] BJ Yun et al., Direct ECC Bypass Phenomena Observed in the MIDAS Test Facility During LBLOCA Reflood Phase, Nuclear Engineering and Technology, October 2002. [3] CH Song et al., Direct Vessel Injection Test Using the MIDAS Test Facility, MIDAS-QLR-009, KAERI, [4] HK CHO et al., Experimental Study for Multidimensional ECC Behaviors in Downcomer Annuli with Direct Vessel Injection Mode during the LBLOCA Reflood Phase, Nuclear Science and Technology, Vol. 42, No. 6, p.549-558, 2005. [5] Stephen Bajorek et al., TRACE V5.0 Developmental Assessment Manual – Appendix B – Separate Effects Tests, US-NRC. [6] Stephen Bajorek et al., TRACE V5.0 Theory Manual, US-NRC. [7] Stephen Bajorek et al., TRACE V5.0 User's Manaul Volume 1: Input Specification, US-NRC.

> Transaction of the Korean Nuclear Society Spring Meeting Jeju, Korea, May 23-24, 2019

