Fission-based Hot Test Production of Mo-99 Using HANARO

Seung-Kon Lee*, Suseung Lee, Myunggoo Kang, Kyungseok Woo, Soon-Bog Hong, Ul Jae Park

Division of Neutron and Radioisotope Application Research, Korea Atomic Energy Research Institute, 989-111

Daedoek-daero, Yuseong-gu, Daejeon, 305-353 Republic of Korea

*Corresponding author: seungkonlee@kaeri.re.kr

1. Introduction

Molybdenum-99 (⁹⁹Mo) and its daughter ^{99m}Tc has been the most commonly used medical radioisotope which covers more than 80% of overall nuclear diagnostics. Majority of commercial-scale ⁹⁹Mo production is based on the fission of ²³⁵U. The ⁹⁹Mo generated from the fission (fission ⁹⁹Mo) exhibits very high specific activity compared with ⁹⁹Mo generated from the other routes: neutron activation or acceleratordriven. [1] These days, international ⁹⁹ Mo supply is unstable due to the frequent and unscheduled shutdowns of aged research reactors irradiating ⁹⁹Mo targets.

In the present, ⁹⁹Mo is imported from abroad, and stable supply of the isotope is important to provide essential diagnostic services. Besides, export of ⁹⁹Mo can be a strong driving force to promote radiation and radioisotope industry of Korea. For the purposes, KAERI is developing LEU-based fission ⁹⁹Mo production process from 2012 to be implemented to the new research reactor (KJRR), which is being constructed in Gijang, Busan, Korea.

Historically, the most ⁹⁹Mo producers have been used highly enriched uranium (HEU) targets so far. However, to reduce the use of HEU in private sector for nonproliferation, all producers are forced to convert their HEU-based process to use low enriched uranium (LEU) targets. Consequently, overall cost for the production of the fission ⁹⁹Mo increases significantly with the conversion of fission ⁹⁹Mo targets from HEU to LEU. It is not only because the yield of LEU is only 50% of HEU, but also because radioactive waste production increases 200%. Therefore, finding optimal treatment of radiowastes from fission ⁹⁹Mo production process become more important. [2, 3]

2.Development Progress of Fission ⁹⁹Mo Process

In 2012, development of fission Mo-99 production process has been initiated. Process development was tested via cold experiments with unirradiated depleted uranium (DU) or low enriched uranium (LEU) targets until 2017.

Today, all industrial-scale producers of ⁹⁹Mo use dedicated targets with a configuration similar to the reactor fuels. Since fuels of early times were generally uranium-aluminum alloy cladded with aluminum shell. KAERI developed plate-type LEU target composed of UAl_x meat dispersed in Al-6061 cladding.

After irradiation of the targets in the research reactor, the targets are dissolved in caustic solution to separate ⁹⁹Mo out of it. Other fission products including unreacted uranium and actinides are removed from the solution, and collected as a solid radiowastes. In the fission Mo-99 production process with caustic digestion, most iodine remains in the liquid phase as negatively charged iodide form. Isotopes of iodine and soluble elements are collected as a liquid radiowastes from the process stream. Radioisotopes of xenon (Xe) and krypton (Kr) are generated from the fission of Uranium. Major products from the production of fission-based radioisotopes are ^{131m}Xe, ¹³³Xe, ^{133m}Xe, ¹³⁵Xe, ^{135m}Xe, ⁸⁵Kr, ^{85m}Kr and ⁸⁷Kr. Emission of radioxenon from the medical radioisotope production is controlled via gaseous waste treatment system with multiple steps of mitigation and confinement. First, process equipment and production hot cells are made as closed-system with leak-tight parts to minimize effluence of Xe from the system. In spite of the leak-tight systems, it is impossible to completely confine Xe in the system. Therefore, proper combination of equipment to reduce the xenon emission is installed in the medical radioisotope production facility. Finally, medical-grade ⁹⁹Mo can be extracted after repeated separation and purification steps. [4, 5, 6]

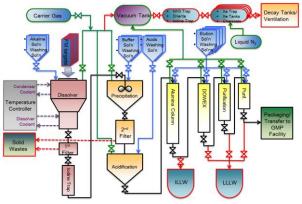


Fig. 1. Scheme for the KAERI's fission Mo-99 process

As a part of the project, KAERI developed technologies to treat radiowastes generated from the ⁹⁹Mo production. It covers generation, transfer, storage and disposal of liquid, gaseous and solid wastes containing various fission products and salts. KAERI developed new technology to facilitate waste treatment by converting sludge-type waste, which is difficult to handle, into independent solid and liquid wastes. Using this scheme, salt concentration in the ILW can be

reduced significantly to make cementation much easier. KAERI developed compact xenon adsorption module using chilled carbon column. Compared with the normal carbon column, KAERI system requires only 1/3,700 carbon in weight or 1/980 in volume for equivalent performance.

3. Result and Discussion

In 2018, hot production test was performed in HANARO site. DU target plates were irradiated in HANARO. And then, the plates were transferred to the hot cells located in the Irradiated Material Examination Facility (IMEF) for the dissolution process. After removal of spent uranium, radioiodines and noble gas from the product, crude ⁹⁹Mo solution was transported to the hot cells located in the Radioisotope Production Facility (RIPF) for the separation. Inventory of Mo-99 in the irradiated target plates were estimated by calculation. Finally, total amount of produced Mo-99 at each production stage was characterized by gamma spectroscopy.

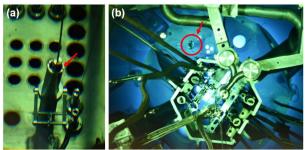


Fig. 2. Irradiation at HANARO. (a) Target capsule and irradiation rig assembly. (b) Loading target rig assembly in the IP5 hole of HANARO.



Fig. 3. Fission ⁹⁹Mo Hot test production systems. (a) System for target dissolution installed in the irradiated material examination facility (IMEF) of KAERI. (b) System for ⁹⁹Mo separation installed in the radioisotope production facility (RIPF) of KAERI.

5. Conclusions

From the hot test production in HANARO in 2018, over 2 Ci of fission ⁹⁹Mo has been successfully produced and separated. The result of the process development will be a first step toward commercial-scale ⁹⁹Mo production in the new research reactor of Korea. KAERI is aiming for the weekly production of

2000 Ci (6-day calibrated) fission ⁹⁹Mo from the KJRR. The amount corresponds to the 100% of domestic demand, and 20% of international market.

REFERENCES

[1] International Atomic Energy Agency, "Non-HEU Production Technologies for Molybdenum-99 and Technetium-99m", IAEA Nuclear Energy Series No. NF-T-5.4, 2013.

[2] International Atomic Energy Agency "Fission Molybdenum for Medical Use", IAEA-TECDOC-515, 1989.

[3] International Atomic Energy Agency, "Management of Radioactive Waste from ⁹⁹ Mo Production", IAEA-TECDOC-1051, 1998.

[4] M. V. Wilkinson, A. V. Mondino, A. C. Manzini, J. Radionanl. Nucl. Chem., Vol. 256, p. 413, 2003.

[5] R. Muenze, G. J. Beyer, R. Ross, G. Wagner, D. Novotny, E. Franke, M. Jehangir, S. Pervez, A. Mushtaq, Sci. and Tech. of Nuclear Installations, 932546, p. 9, 2013.

[6] S. Dittrich, Sci. and Tech. of Nuclear Installations, 514894, p. 9, 2013.