



# A sensitivity study on the initial RCS conditions in the major Non-LOCA accidents

## Ilsuk Lee\*, Sarah Kang, Youngseok Bang

Korea Institute of Nuclear Safety (KINS), 62 Gwahak-ro, Yuseong-gu, Daejeon 34142, Republic of KOREA \*islee@kins.re.kr

#### Introduction

- \* KINS has been developing a Non-LOCA regulatory audit methodology, which consisted of advantages from both the conservative method and BEPU approach.
- Non-LOCA transients have consisted of a lot of events and simple phenomena during a short transient time in contrast to LOCA. Thus, totally applying to many uncertainty parameters for Non-LOCAs is not efficient in terms of costs and efforts.
- As the way to ensure safety margin among the several advantages, the use of conservative initial RCS operating conditions derived from the sensitivity study is one of the best ways to ensure enough safety margin in the field of safety assessment.
- This study is aimed to give valuable insight on the impact of acceptance criteria under varying initial RCS conditions using the sensitivity study.

#### Test matrix of a sensitivity study

- Sensitivity parameters are selected as pressurizer pressure, cold leg temperature, and core inlet flow rate and applied to non-LOCA accidents such as main steam line break (MSLB), main feed line break (MFLB), locked rotor (LR), steam generator tube rupture (SGTR) of APR1400.
- Sensitivity ranges were based on limited operation and condition (LCO) of technical specifications. All of the results calculated by MARS-KS system code focused on acceptance criteria such as RCS pressure, secondary system pressure, and departure from nucleate boiling ratio (DNBR) in representative transients of non-LOCAs.
- ❖ Table 1 shows the range of sensitivity based on LCO, each of which is applied to major transients. 72 cases of steady-state calculations, including base cases, were performed to meet steady-state conditions for each major transient.
- ❖ After performing steady-state calculations, the same transient scenario was applied to 72 normal conditions respectively to obtain 216 results in all three acceptable criteria: nuclear boiling ratio, maximum RCS pressure, and maximum SG pressure.

Table 1. Sensitivity range on the RCS conditions

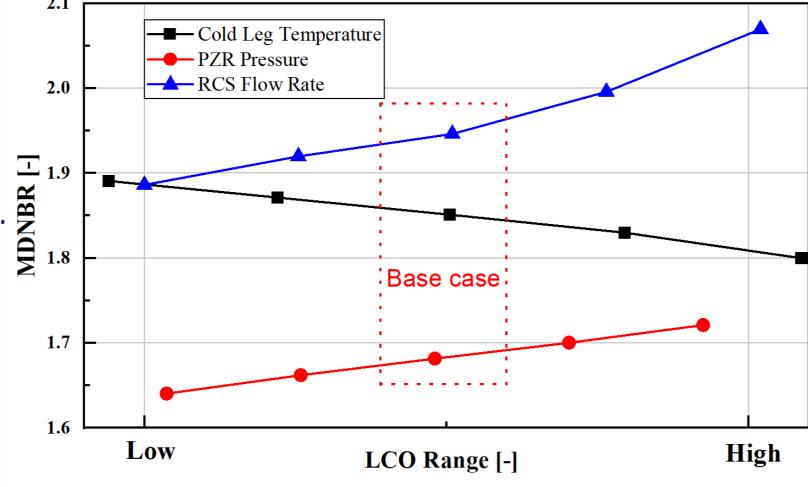
Accident	Parameter	Range	Criteria
MSLB, MFLB, LR, SGTR	PZR Pr.(Bar)	152 ~ 158	Min. DNBR, Max. RCS Pr., Max. SG Pr.
	RCS Temp.(°C)	289.1 ~ 293.6	
	RCS Flow(kg/s)	21,000~ 24,200	

#### Trends on the initial RCS conditions

- System behaviors of major transients, which are MSLB, MFLB, LR, SGTR, were analyzed using MARS-KS code to develop the trend map on the RCS initial conditions.
- Fig. 1. DNBR trends at the SGTR Because there was a great deal of analysis

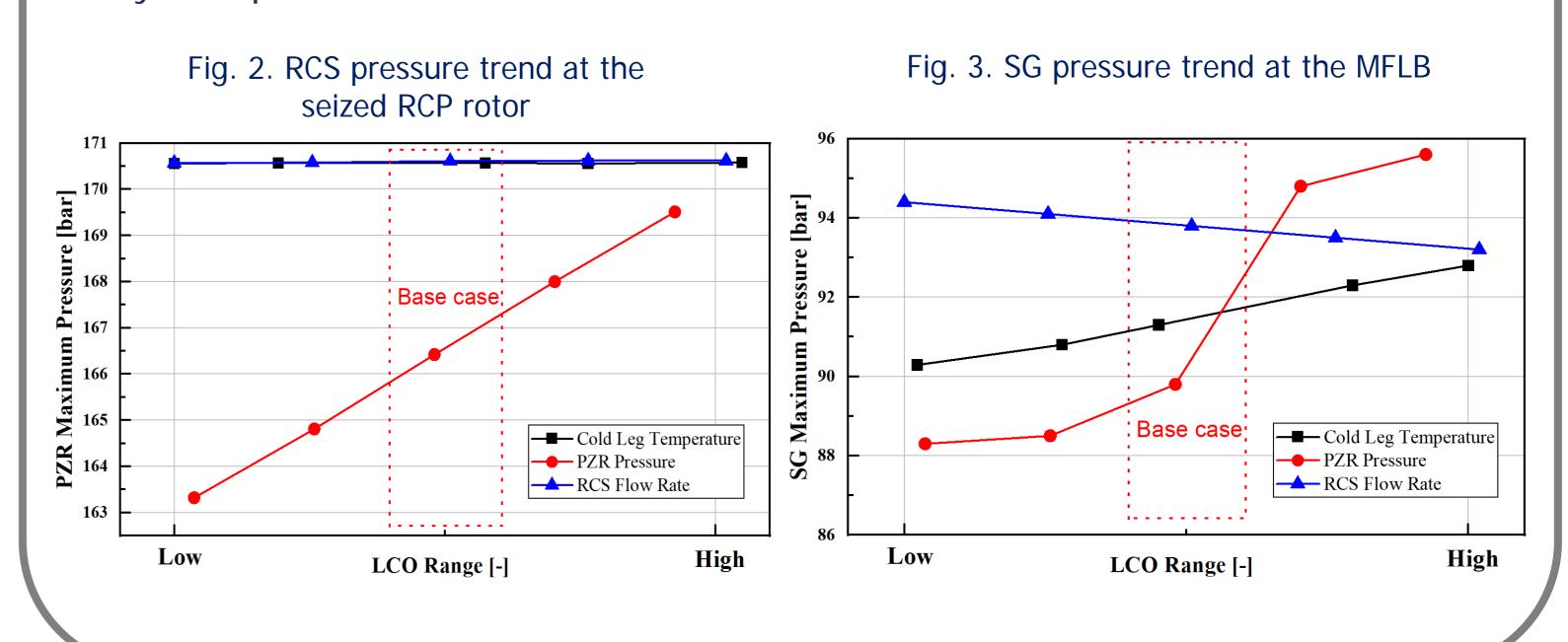
results on the transients, this paper described focusing on the results showing the trend clearly with respect to the minimum DNBR, maximum RCS, and SG pressure. \( \mathbb{g} \)

SGTR accident represented the impact of variance on the RCS initial conditions in terms of minimum DNBR. The trend of minimum DNBR was observed



to be lower at the condition not only the higher cold-leg temperature was but also the lower PZR pressure and RCS flow rate were. Figure 1 illustrated the DNBR trend in the RCS operating parameters.

❖ Additionally, the trend of secondary pressure at the MFLB dramatically varied as gradually increasing PZR initial pressure, and the other parameters somewhat affected that of MFLB. Figure 3 represented the variance of the secondary system pressure at MFLB.



## **Development of trend map**

❖ Although sensitivity studies have been conducted using system code so far, the impact trend has not been consistent due to many design changes and unknown parameters affecting safety outcomes, even though power plants of type, for example, APR1400 and OPR1000, have analyzed the same initial state. Thus, the development of a tendency map to easily understand the degree of conservatism in each major transient can be very useful for regulatory staff or designers.

Fig. 4. Trend map on the DNBR in all transient

LCO Range of RCS Initial Flow Rate[-]

**→** MFLB

**-**▼-SGTR

**→**LR

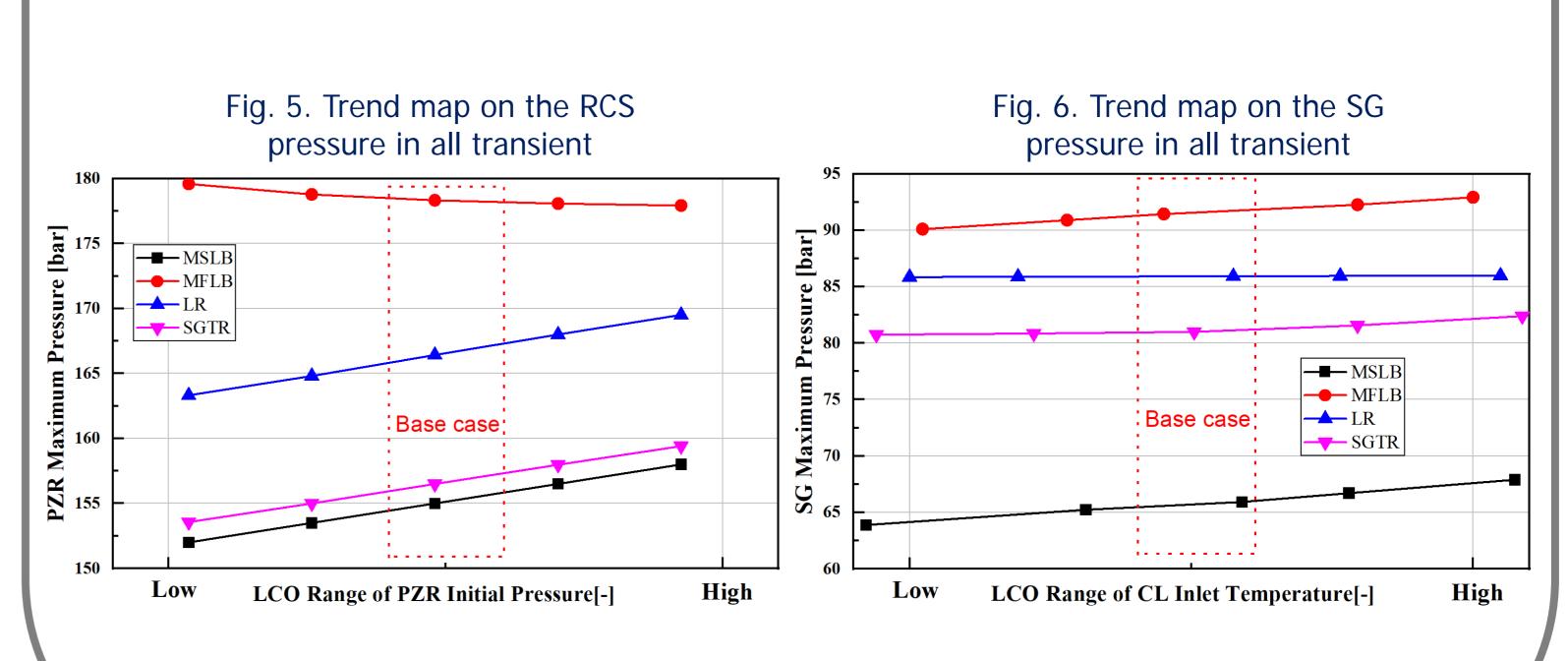
- Figures 4, 5, and 6 represented some of the trend maps on the minimum DNBR, maximum RCS, and SG pressure during all transients being studied in this research.
- ❖ In Figure 4, it was observed that in the case of a locked rotor accident, there was little effect on DNBR despite the change in the flow rate of the core inlet coolant.

The locked rotor is expected to have

the largest asymmetry for core inlet flow rate than other accidents analyzed,

but the system code is very limited to this analysis.

Therefore, it is believed that the use of sub-channel code in detail is necessary to analyze the tendency of minimum DNBR to the change in core inlet flow rate of the locked rotor.



### Conclusions

This study was conducted using MARS-KS to investigate how changes in RCS initial conditions in major non-LOCAs affect regulatory acceptance criteria. Using the results of each accident, a trend map was developed to easily understand the effects of the three acceptance criteria. However, it was found that the use of sub-channel code was also necessary to investigate changes in core flow rates. The study is expected to be completed after analyzing variable sensitivity to secondary system variables and fuel performance.