

## Development of the Program on Radioactivity Evaluation for SF Dry Storage

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### 1. Introduction

The radioactivity module included in the SCALE means an automatic module that can evaluate the inventory with Nuclear Power Plants during transporting the SNF(Spent Nuclear Fuel). SCALE program currently is a step to perform the verification and validation due to completion of design and development and its distributions 3rd version by this time.[1]

In this study, calculations for multiple fuels and automatically derivation of a report form for submitting to regulatory institute, the radiation dose considering the transport of multiple fuels, pre-verification for acceptance criteria for transport and data management system for SF emitted from the nuclear reactor.

The SCALE program evaluated to help operator to manage program and prevent errors on the transportation standards.

### 2. Methods and Results

The radioactivity evaluation module was developed to select the SCALE6.1-OrigenArp Express/Detail mode and express combustion profile applicable to Detail mode. Also, this module could implement a continuous calculation to the multiple fuels which are selected by users and have a relation with safety information for UCMS. The evaluation results for the radioactivity on the multiple fuels could be automatically printed out report format to be able to provide the final report with regulatory institute. Fig. 1 shows a main menu for radioactivity evaluation that can be connected with SCALE module.

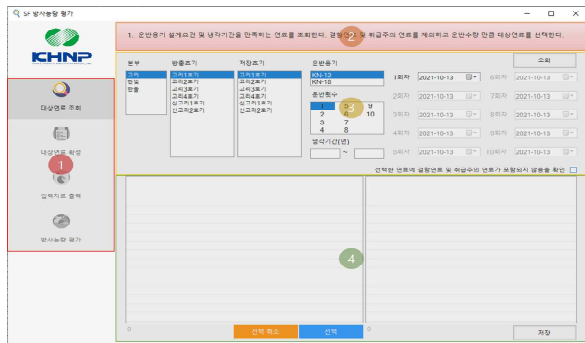


Fig. 1. Configurations for radioactivity evaluation module.

A function of SNF transportation on the main menu screen means to radioactivity evaluation. The radioactivity evaluation module was included such as

“①radioactivity evaluation menu“, “②information message“, “③input menu“, “②evaluation result” was activated. search for a target fuels, determination for a target fuels, deviation for input materials and evaluation radioactivity. SCALE program can be calculated multiple fuels and automatically derive a report form for submission to regulatory agencies.

#### 2.1 Searching and determining for targets

The functions of target fuel for searching a target fuel for radioactivity assessment based on the spent fuel safety information stored in the UCMS. In case of inquiry, the headquarters, discharge and storage units can limit the target to those suitable for the conditions of the relevant power plant among all fuels stored in UCMS. The transport container can be limited to fuel that meets the design requirements of KN-12 and KN-18. The number of transports can be selected up to 10 times, and the transport date can be designated for each cycle as many times as selected. It made based on at least 7 years and there is no upper limit. It established a transport strategy by calculating the radiation dose considering the transport of multiple fuels.

As shown in Fig. 2, the target fuels are sorted in 'ascending order' based on the fuel number.

순번	연료번호	연료유형	연료종류	초기유탄중량(g)	초기중속도(w/o)	방출연속도(MWD)	남각기간(년)	
<input checked="" type="checkbox"/>	1	KLJZF01	17WH	KOFA	4416700	3.5277	40995.0	26.9
<input checked="" type="checkbox"/>	2	KLJZF02	17WH	KOFA	4411140	3.524	41064.0	26.9
<input type="checkbox"/>	3	KLJZF03	17WH	KOFA	4414620	3.526	41379.0	26.9
<input type="checkbox"/>	4	KLJZF04	17WH	KOFA	4399990	3.5141	41191.0	26.9
<input checked="" type="checkbox"/>	5	KLJZF05	17WH	KOFA	4403430	3.5163	31646.0	26.9
<input type="checkbox"/>	18	KLJZF18	17WH	KOFA	4413600	3.5069	31657.0	26.9
<input type="checkbox"/>	19	KLJZF19	17WH	KOFA	4411870	3.5053	41450.0	26.9
<input type="checkbox"/>	44	KLJZF44	17WH	KOFA	4390200	3.4506	42330.0	26.9
<input type="checkbox"/>	45	KLJZF45	17WH	KOFA	4389750	3.4505	42326.0	26.9
<input checked="" type="checkbox"/>	46	KLJZF46	17WH	KOFA	4390120	3.4514	42662.0	26.9
<input checked="" type="checkbox"/>	47	KLJZF47	17WH	KOFA	4404550	3.5061	39932.0	22.8
<input checked="" type="checkbox"/>	48	KLJZF48	17WH	KOFA	4403650	3.4944	36394.0	22.8
<input checked="" type="checkbox"/>	49	KLJZF49	17WH	KOFA	4412600	3.8009	37652.0	25.5
<input checked="" type="checkbox"/>	50	KLJZF50	17WH	KOFA	4429410	3.7951	36360.0	25.5
<input checked="" type="checkbox"/>	51	KLJZF51	17WH	KOFA	4418210	3.7974	38539.0	24.2
<input checked="" type="checkbox"/>	52	KLJZF52	17WH	KOFA	4419340	3.7945	36922.0	25.5
<input checked="" type="checkbox"/>	53	KLJZF53	17WH	KOFA	4418420	3.7943	38514.0	24.2
<input checked="" type="checkbox"/>	54	KLJZF54	17WH	KOFA	4418220	3.7913	37216.0	25.5
<input type="checkbox"/>	55	KLJZF55	17WH	KOFA	4416770	3.7896	38533.0	24.2
<input type="checkbox"/>	56	KLJZF56	17WH	KOFA	4416190	3.7899	37433.0	25.5

Fig. 2. Configurations of result for output target

Among the searched fuels, the user selects the target fuel for which the radiation dose evaluation can be performed. For example, if 'KN-12' is selected for 'carrier', a total of 12 bundles are checked in the order of the sorted fuel, including the fuel checked in the checkbox, among the sorted fuels. As checking a fuel No. 3 in turn, No. 3 to No. 14 are checked at all.

The nuclear fuel emission unit or storage unit has a function of selecting only the nuclear fuel for which radioactivity is to be evaluated among the total fuel

stored in the UCMS. Additional selected SNF is selected as a total of 12 bundles or 18 bundles according to the transport container.

### 2.2 Output for Radioactivity evaluation

As searching for the target fuel, you can check the fuel that meets the design requirements and cooling period. Excluding defective fuels or fuels with handling precautions, target fuels with a large transport quantity can be selected. For the discharge unit and the storage unit, the selectable units are derived differently for each power plant selected. The finally selected fuels are sorted in 'ascending order' by classifying the colors for each cycle as shown in Fig. 3.

순번	연료번호	연료유형	연료종류	초기우라늄량(g)	초기농도(w/o)	방출연소도(MWD/L)	냉각기간(년)
6	KLUZF06	17WH	KOFA	440633.0	3.5124	31986.0	25.7
7	KLUZF07	17WH	KOFA	441173.0	3.5097	41106.0	25.7
8	KLUZF08	17WH	KOFA	441609.0	3.5085	42222.0	25.7
9	KLUZF09	17WH	KOFA	442148.0	3.5099	31517.0	25.7
10	KLUZF10	17WH	KOFA	442348.0	3.5108	41329.0	25.7
11	KLUZF11	17WH	KOFA	442601.0	3.5111	42376.0	25.7
12	KLUZF12	17WH	KOFA	441451.0	3.5093	41274.0	25.7
13	KLUZF13	17WH	KOFA	442592.0	3.5111	33467.0	25.7
14	KLUZF14	17WH	KOFA	441389.0	3.5091	33452.0	25.7
15	KLUZF15	17WH	KOFA	441511.0	3.5086	31759.0	25.7
16	KLUZF16	17WH	KOFA	441243.0	3.5076	31661.0	25.7
17	KLUZF17	17WH	KOFA	441385.0	3.5083	41522.0	25.7
20	KLUZF20	17WH	KOFA	441220.0	3.5055	33608.0	25.9
21	KLUZF21	17WH	KOFA	441011.0	3.5042	41741.0	25.9
22	KLUZF22	17WH	KOFA	440500.0	3.5153	31925.0	25.9
23	KLUZF23	17WH	KOFA	440167.0	3.518	31877.0	25.9
24	KLUZF24	17WH	KOFA	440815.0	3.5074	42609.0	25.9
25	KLUZF25	17WH	KOFA	439972.0	3.5155	33871.0	25.9
26	KLUZF26	17WH	KOFA	440166.0	3.5105	42246.0	25.9
27	KLUZF27	17WH	KOFA	440294.0	3.5056	41505.0	25.9
28	KLUZF28	17WH	KOFA	440977.0	3.5055	41576.0	25.9

Fig. 3. Evaluation results for target spent fuel.

As shown in Fig. 4, the target fuel confirmation menu evaluates whether the fuel selected in the previous step satisfies the storage requirements of the unit to be transported and stored (hereinafter referred to as the receiving unit), and the final radioactivity evaluation for the fuel that meets the storage requirements.

연료번호	연료번호	초기우라늄량(w/o)	E_4	E_3	E_2	E_1	E_0	방출연소도(MWD/L)	냉각기간(MWD/L)	운행일수	운행일수	평가결과
17WH	KOFA	4.9989	0	175.921	2149.61	21051.9	20417.7	46570	42882.0	1	2021-10-13	합격
17WH	KOFA	3.4913	0	175.921	2149.61	21051.9	20417.7	27409	33761.0	1	2021-10-13	합격
17WH	KOFA	3.4939	0	175.921	2149.61	21051.9	20417.7	27442	20291.0	1	2021-10-13	합격
17WH	KOFA	3.4980	0	175.921	2149.61	21051.9	20417.7	27479	43139.0	1	2021-10-13	합격
17WH	KOFA	3.4989	0	175.921	2149.61	21051.9	20417.7	27480	43070.0	1	2021-10-13	합격
17WH	KOFA	3.4982	0	175.921	2149.61	21051.9	20417.7	27487	33780.0	1	2021-10-13	합격
17WH	KOFA	3.5089	0	175.921	2149.61	21051.9	20417.7	27508	33860.0	1	2021-10-13	합격
17WH	KOFA	3.5049	0	175.921	2149.61	21051.9	20417.7	27533	43106.0	1	2021-10-13	합격
17WH	KOFA	3.5086	0	175.921	2149.61	21051.9	20417.7	27530	28340.0	1	2021-10-13	합격
17WH	KOFA	3.5039	0	175.921	2149.61	21051.9	20417.7	27570	33818.0	1	2021-10-13	합격
17WH	KOFA	3.4991	0	175.921	2149.61	21051.9	20417.7	26987	42270.0	1	2021-10-13	합격
17WH	KOFA	3.4571	0	175.921	2149.61	21051.9	20417.7	26972	42460.0	1	2021-10-13	합격
17WH	KOFA	3.4557	0	175.921	2149.61	21051.9	20417.7	26954	42449.0	2	2021-12-01	합격
17WH	KOFA	3.4506	0	175.921	2149.61	21051.9	20417.7	26889	42200.0	2	2021-12-01	합격
17WH	KOFA	3.4506	0	175.921	2149.61	21051.9	20417.7	26887	42200.0	2	2021-12-01	합격
17WH	KOFA	3.4514	0	175.921	2149.61	21051.9	20417.7	26889	42620.0	2	2021-12-01	합격
17WH	KOFA	3.5061	0	175.921	2149.61	21051.9	20417.7	27538	36302.0	2	2021-12-01	합격
17WH	KOFA	3.4944	0	175.921	2149.61	21051.9	20417.7	27449	36384.0	2	2021-12-01	합격
17WH	KOFA	3.5009	0	175.921	2149.61	21051.9	20417.7	31339	37652.0	2	2021-12-01	합격

Fig. 4. Configurations for target determination.

Duplicate selections are applied to all fuels subject to evaluation if they are selected only once before evaluation. The evaluation result can be checked in the output file, and the radiation dose evaluation result can be derived based on the last information of the input data. In particular, the evaluation result in the form of a procedure is automatically generated as a file. After the

radiation dose evaluation, the result of the total radiation dose for each container is derived, and the safety criterion is “less than  $3.75 \times 10^6$  Ci” in the case of KN-12 and “less than  $6.018 \times 10^6$  Ci” in the case of KN-18, evaluated as satisfactory. A pre-verification system was applied to ensure that the selected fuel satisfies the acceptance criteria for transport containers to prevent errors due to improper loading. Furthermore, Data management system (Fig.4) was facilitated by establishing a DB for SF emitted from the nuclear reactor.

### 3. Conclusions

The main functions applied to this program are as follows. 1. Calculate multiple fuels and automatically derive a report form for submission to regulatory agencies. 2. It is possible to establish a transport strategy in advance by calculating the radiation dose considering the transport of multiple fuels. 3. A pre-verification system was applied to ensure that the selected fuel satisfies the acceptance criteria for transport containers to prevent errors due to improper loading. 4. Data management was facilitated by establishing a DB for SF emitted from the nuclear reactor. 5. It was made possible to verify in advance whether it conforms to the loading curve of the acquired unit. This program was evaluated to be able to help increase operator convenience and prevent errors in transportation standards.

### REFERENCES

- [1] Korea Hydro & Nuclear Power Co., LTD, “Characteristic Inspection and Evaluation of spent fuel for Dry Storage”.
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