

Corrosion Characteristics of FeCrW Model Alloys in 360°C High Purity Water



Jun Yeong Jo, Chi Bum Bahn* School of Mechanical Engineering, Pusan National University * Corresponding author: bahn@pusan.ac.kr

Introduction

- One of the main candidates for fusion power reactors' structural materials are Reduced Activation Ferritic/Martensitic Steels.
- These iron-based RAFM steels are mainly composed of 8~9% Cr and 1~2% W and replaced high activation minor elements from conventional FM steels to low activation elements like V, Ta etc.[1].
- Water is one of the main candidates for coolant of various fusion reactor designs, and the studies with corrosion characteristics of FM steels showed the Cr concentration dependency on corrosion resistance.

- With reducing Cr concentration below 12%, the corrosion rate increased rapidly, but considering the other mechanical properties like fracture toughness, it appears that the 9% Cr is the most promising Cr concentration[2,3].
- Although RAFM steels are quite vulnerable to PWR conditions due to low Cr concentration, it seems that water-cooled components of RAFM steels can be safely operated, at least under proper coolant chemistry control[4].
- As for the first step of the study, we have investigated corrosion properties of three model alloys in stagnant 360°C high purity water conditions.

Literature Data Analysis

- Before conducting corrosion tests, we collected weight change data from prior studies conducting corrosion tests with RAFM steels or other FM steels, as shown in Fig.1 [5-12].
- As the test temperature increases, the weight change rate becomes higher. It appears that the overall weight gain behavior follows the parabolic law more significantly as the test temperature increases.
- Under flowing water conditions, the weight loss was observed. It seems that the weight loss is suppressed as the dissolved oxygen(DO) concentration of the flowing water becomes high (see Fig. 1) [12].
- identify insufficient data points of conducted test conditions, we collected corrosion test conditions that had been reported on the prior studies and arranged like Fig. 2.

Experimental Methods

Experimental conditions

Alloys chemical compositions (wt%)

Parameter	Value		Fe	Cr	W
Material	Fe – 9, 12% Cr – 0, 1% W	Fe12Cr1W	Bal.	11.4 %	1.04 %
Temperature	360°C	Fe9Cr1W	Bal.	9.2 %	1.07 %
Water Chemical Condition	Dissolved Oxygen(DO) : Deaerated, 1ppm	Fe9Cr	Bal.	8.8 %	_
Water Flow	Static				
Exposure Time	50, 100, 150, 300h				



Fig.1. Weight change results from prior corrosion tests studies of FM steels [5-12].

Fig.2. Corrosion test conditions of prior studies. The Orange dots are the conditions of this study.







Results



- Until 100h of corrosion test in DO level of 1ppm, the weight change rate does not seem to be affected by Cr concentrations (see Fig. 3).
- After 50h of corrosion test conducted in DO level of 1ppm, the surface oxides were examined by SEM (see Fig. 4).
- The blocky oxides were formed on the
 - surface of the specimens.
- There was no significant difference in

Fig.3. Weight change results of FeCrW model alloys in DO 1ppm condition

surface oxide morphology until the 50-

hour corrosion test.

Fig.4. Surface oxide morphology results of FeCrW model alloys

x10k

(SEM SE image). Test conducted in DO level of 1ppm for 50h.

Future Work

- The continuous corrosion tests up to 300h is on going and additional corrosion test in DO level of 0ppm will be conducted.
- The results of the corrosion tests will be combined and compared with the prior corrosion studies.
- In addition to weight change measurement and surface morphology analysis, measurement of dissolution rate of the alloys, XRD surface analysis and oxide film cross-section analysis will be conducted to study FeCrW model alloys' corrosion characteristics in high temperature, high purity water.

References

- [1] 이창훈, 문준오, 박성준, 이태호, 홍현욱, 신찬선, ... & 김형찬, 핵융합용 저방사화 철강소재 개발 현황. 재료마당, 32(6), 4-15, 2019
- [2] Misawa. T., Hamaguchi. Y., & Saito. M., Journal of Nuclear Materials, 155, 749-753, 1988.

x1k

- [3] Kimura. A., Kayano. H., Misawa, T., & Matsui. H., Journal of nuclear materials, Vol.212, p. 690-694, 1994.
- [4] De Meis. D., Corrosion resistance of RAFM steels in pressurized water for nuclear fusion applications, 2017.
- [5] Shiba, K., Hishinuma, A., Tohyama, A., & Masamura, K., Properties of low activation ferritic steel F82H IEA heat. Interim report of IEA round-robin tests. 1, 1997.
- [6] Allen, T. R., Chen, Y., Tan, L., Ren, X., Sridharan, K., & Ukai, S., Corrosion of candidate materials for supercritical water-cooled reactors. In Proceedings of the 12th International Conference on Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors (pp. 1397-1407), 2005.
- [7] Ren, X., Sridharan, K., & Allen, T. R, Journal of Nuclear Materials, 358(2-3), 227-234, 2006.
- [8] Lo Piccolo, E., Torella, R., Terranova, N., Di Pace, L., Gasparrini, C., & Dalla Palma, M., Corrosion and Materials Degradation, 2(3), 512-530, 2021.
- [9] Hirose, T., Shiba, K., Enoeda, M., & Akiba, M., Journal of nuclear materials, 367, 1185-1189, 2007.
- [10] Kanai, A., Kasada, R., Nakajima, M., Hirose, T., Tanigawa, H., Enoeda, M., & Konishi, Journal of Nuclear Materials, 455(1-3), 431-435, 2014.
- [11] Yamanouchi, N., Tamura, M., Hayakawa, H., Hishinuma, A., & Kondo, T., Journal of nuclear materials, 191, 822-826, 1992.
- [12] Nakajima, M., Hirose, T., Tanigawa, H., & Enoeda, M., Plasma Fusion, 2015.