# Simulation of 50% DVI Line SBLOCA in the ATLAS Integral Test Facility Using MARS-3D Code

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## 1. Introduction

For a thermal-hydraulic integral effect test facility, ATLAS (Advanced Thermal-hydraulic Test Loop for Accident Simulation) [1], a 50% break of a DVI line of the APR1400 (named SB-DVI-07) [2] was carried out and selected as the 50<sup>th</sup> international standard problem exercise (ISP-50) by OECD/CSNI on November, 2008 due to its technical importance.

Since the multi-dimensional peak cladding temperature (PCT) behavior in the core was observed in the SB-DVI-07, the 3-dimensional (3-D) core model is used for the MARS-3D [3] simulations.

# 2. MARS-3D Modeling

MARS-3D (MARS-KS version 002) used for the present simulation is started with a consolidated code of RELAP5/MOD3 and COBRA-TF. Then, MARS-KS has been equipped with multi-D component to model a three dimensional component with Cartesian and cylindrical coordinate system.

Overall nodalization diagram is shown in Fig. 1. For this nodalization the number of volumes, junctions and heat structures are 1096, 2435, and 406, respectively.



Fig. 1. Nodaization of the ATLAS facility.

#### 2.1 Multi-Component Model

The reactor pressure vessel component is modeled as 3-D components (as shown in Fig. 2): 30 axial, 5 radial, and 6 azimuthal sectors. The active core region has 10 axial sub-volumes, each 4-heater group is located at  $1^{st}$ ,  $2^{nd}$ ,  $3^{rd}$ , and  $4^{th}$  radial ring, respectively.



Fig. 2. MARS multi-D component for reactor vessel.

#### 2.2 Bypass Modeling

- 2 hot leg bypass lines connecting the downcomer region and the inlet of 2 hot legs
- 2 upper head bypass lines connecting the upper head and the neighboring downcommer region

#### 2.3 Heat Structure Model

Every heat structure is modeled to account the stored energy and heat loss to the ambient. For the active core region 24 heat structures are modeled to each 4-heater group

- 1 heat structure for each sector of the ring
- Uniform heat capacity for each heater rod and heat loss is considered : 3.98 kW/rod.

Table I: Number of heater rods for each sector / ring



#### 2.4 Choked Flow Model

Trapp-Ransom model is used to match to test break flow.

#### 2.5 Turbine Boundary Pressure

Turbine boundary pressure is set to ambient pressure. And turbine flow area is adjusted to get a test flow rate.

#### 3. Calculation Results

## 3.1 Steady-state Results

The 3-D model in the reactor vessel component by MARS-3D well predicts the steady state results of ISP-50 as shown in Table II.

Table II	· Com	parison	of	steady	state	results
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Parameter	Measured value	MARS	Instruments	
Primary system				
- Core power (MW)	1.535	1.535	Including heat loss	
- PZR Pressure (MPa)	15.596	15.596	PT-PZR-01	
- Core inlet temp. (K)	563.2	565.5	TF-LP-02G18	
- Core exit temp.(K)	598.2	599.2	TF-CO-07-G14, G18, G21, G25	
11-11-1-1-1-1-00	597.6	598.2	TF-HL1-03A	
- Hot leg temp. (K)	598.5	598.3	TF-HL2-03A	
	565.4	563.2	TF-CL1A-04A	
C-141	565.3	563.2	TF-CL1B-04A	
- Cold leg temp. (K)	565.0	563.2	TF-CL2A-04A	
	565.2	563.2	TF-CL2B-04A	
	2.2	1.964	QV-CL1A-01B	
- RCS flow rate (kg/s)	±5%	1.964	QV-CL1B-01B	
		1.966	QV-CL2A-01B	
		1.966	QV-CL2B-01B	
- Pressurizer level (m)	3.32	3.38	LT-PZR-01	
Secondary system	(SG1/SG2)			
- Pressure (MPa)	7.83/7.83	7.91/7.91	PT-SGSD1-01/PT-SGSD2-01	
- Steam temp. (K)	568.6/568.8	567.4/567.4	TF-SGSD1-03/TF-SGSD2-03	
- FW temp. (K)	507.6/508.4	507.6/508.4	TF-MF1-03/TF-MF2-03	
- FW flow rate (kg/s)	0.431/0.435	0.45/0.45	QV-MF1-01/QV-MF2-01	
- Water level (m)	2.03/1.97		LT-SGSDDC1-01/LT- SGSDDC2-01	
ECCS				
- SIT pressure (MPa)	4.19/4.21/4.23	4.19/4.21/4.23	PT-SIT1,2,3-02	

#### 3.2 Transient Results

transient behavior of Initial and cladding temperatures is well predicted except the over-cooling of the cladding temperatures at the 1<sup>st</sup> and 2<sup>nd</sup> measurement location at the end of the test (Fig. 3). However the 3-D effect of PCTs in the core region is not captured by the MARS-3D, whereas the heater rods located in the 4<sup>th</sup> quadrant of the cross sectional view showed a higher increase in the PCT than those in other quadrants from the experimental results. The break flow during the two-phase discharge is under-predicted, while the plateau of the primary pressure is not predicted during this period. Accumulated mass of break outflow is under-estimated (Fig. 4) due to the under-estimation of break mass flow rate during the two-phase discharge. As we can see in Fig. 5, the loop seal clearing is predicted in all intermediate legs in calculation results, whereas the loop seal were cleared only in CL-1A and CL-1B (2 out of total 4 cold legs) intermediate legs in experiment.











#### 4. Conclusions

The 3-D model in the reactor vessel component by MARS-3D well predicts the steady state results of ISP-50. However present MARS-3D simulation cannot show the non-uniformity of PCTs. The break flow during the two-phase discharge is under-predicted, while the plateau of the primary pressure is not predicted during this period. The loop seal clearing is predicted in all intermediate legs in calculation results, whereas the loop seal were cleared only in CL-1A and CL-1B intermediate legs in experiment

For the future study the optimization of the number of multi-components (900 cells in the present model) should be investigated to enhance the calculation efficiency (about 91,000 s of CPU time is used for 2,200 s of the transient simulation).

#### REFERENCES

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[3] B.D. Chung et. al, "MARS CODE MANUAL VOLUME II: INPUT REQUIREMENTS", KAERI/TR-2811/2004.