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Preliminary Estimates of Nuclear Weapon Potential in North Korea's New ELWR

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Introduction



• 2016년 – 2017년 (핵보유 시도)

- 4차 핵실험 (16.1월)
- 5차 핵실험 (16.9월)
- 6차 핵실험 및 ICBM 발사 (17.9월)

• 2018년 (북핵 일부 합의)

- 판문점 선언 (4.27)
- 싱가포르, 북미정상회담 공동성명 (6.12)
- 평양공동선언 (9.19)
- ・2019년
 - 하노이, 제2차 북미정상회담 (2.27-28) -> <mark>결렬</mark>
- 2021년 이후, 영변 핵시설 재가동

북미협상: 북미 비핵화 실무협상 에 엇갈린 양국 입장

2019년 10월 6일



북한대표단을 태운 차량이 미국과 회담장소인 스톡홀름 인근 리딩고 섬으로 향하고 있다

베트남 하노이 2차 정상회담 이후 약 7개월여 만에 열린 스웨덴 북-미 비핵화 실무협 상이 사실상 결렬됐다.

앞서 북한 김명길 외무성 순회대사와 미국 스티븐 비건 국무부 대북특별대표가 각각 이끄는 북-미 협상단은 5일(현지 시간) 스웨덴 스톡홀름 북동쪽 리딩고 섬에서 비핵 화 협상을 벌였다.

Figure 1. 북미협상 결렬^[1]



- The Nyongbyon nuclear scientific research center has the IRT-2000
 - research reactor, a 5 MWe reactor, and a 100 MWth reactor.



Figure 2. 북, 영변 주요 핵시설^[2]



- The Experimental Light Water Reactor (ELWR) was recently
 - constructed at the Nyongbyon site in the 2010s.



Figure 3. Development of pipelines at the ELWR from 2010 to 2013^[3]

Introduction



• This new ELWR reportedly began operations in October 2023.



Figure 3. Thermal image over Yongbyon Nuclear Scientific Research Center from October, 2023. Blue (lower heat) -> Red (higher heat)^[3]

Reactor Modeling

• The 5 Megawatt-electric Reactor

- Power: 25 MWth
- Type: Magnox (Graphite-moderated gas-cooled reactor)
- Fuel: 50 tons, metallic natural-uranium (²³⁵U 0.72 wt%)
- From 1986, producing Weapon-Grade Plutonium

Separated WG-**Operation and Residence: Amount.Spent** Reprocess fuel Removed shutdown Duration avg. burnup Pu Op. 1986-1989 Less than or equal to Shutdown 1989 (70-3 years (Unknown) Unknown Unknown 2 kg100 days) Op. 1989–1994 Unknown (~650 Full core: 50 tons U 2003.01-06 20-30 kg Shutdown 1994 MWd/t) Op. 2003-2005 Shutdown 2005 (~70 2 years (330 MWd/t)Full core 2005.06-12 10–14 kg days) Op. 2005-2007 1+ year (Less than Full core 2009 ~8 kg Shutdown July 2007 200 MWd/t) Op. 2013-2015 2 years (intermittent: Likely full core 2016 5.5–8 kg Shutdown 2015 Uncertain burnup) Op. 2016 In Reactor

Table I: Operation history of 5 MWe Reactor and WG-Pu production estimations^[4]



total 45 ~ 62 kg

Reactor Modeling



- The new Experimental Light-Water Reactor
 - Power: 100 MWth
 - Type: VVER-440 (Russia's PWR using hexagonal fuel bundles)
 - Fuel: 4 tons, 3.5 wt% enriched UO₂^[5]
 - Constructed since 2010 & Tested the cooling water system (July 2022)
 - Begun operating since October 2023



Figure 4. A significant amount of water discharge from the ELWR (Left: October 4, 2023 / Right: December 10, 2023)^[6]



• Oak Ridge Isotope GENeration (ORIGEN) module in SCALE code

- Point-depletion (0-D) code that calculate time-dependent concentrations, activities, and radiation source terms for a large number of isotopes simultaneously generated or depleted by neutron transmutation, fission, and radioactive decay
- Neutron spectrum-dependent libraries are created from interpolation of existing reactor libraries in SCALE code, using Automated Rapid Processing (ARP) module.
- Depletion calculations are used the Chebyshev Rational Approximation Method (CRAM) in solving the Bateman equation.
- For post-processing, OPUS module shows calculated isotopics and spectra to be sorted, ranked, and converted to other units.



- Weapon-Grade Plutonium (WG-Pu) estimation
- WG-Pu is defined as plutonium with a high content of the fissile isotope (²³⁹Pu)
- **Pu quality** = $\frac{\text{fissile Pu isotopes mass } (^{239}\text{Pu})}{\text{total Pu isotopes mass } (\text{total Pu})} \ge 93 \text{ wt\%}$
- The Pu quality monotonically decreases due to the preferential fission reactions of ²³⁹Pu.



Figure 5. Comarison of plutonium production and quality over depletion by reactor types (Left: VVER / Middle: Magnox / Right: Pu quality)



Depletion time

- VVER: 116 days
- Magnox: 1390 days

• Burnup

- VVER: 2900 MWd/tU
- Magnox: 695 MWd/tU

• Pu production

- VVER: 5.94 kg
- Magnox: 30.4 kg

Grams of WG-Pu per MWd

- VVER: 0.512 g/MWd
- Magnox: 0.876 g/MWd

	VVER	Magnox
Thermal power (MWt)	100	25
Initial mass of uranium (tons)	4	50
Depletion time at which Pu quality becomes 93 wt% (days)	116	1390
Burnup (MWd/tU)	2900	695
WG-Pu production (kg)	5.94	30.4
Grams of WG-Pu per MWd (gPu/MWd)	0.512	0.876
Separated WG-Pu (kg)	3.74 ~ 4.28	19.2 ~ 21.9
Annual WG-Pu production (kg/year)	11.8 ~ 13.5	5.04 ~ 5.75
Number of nuclear weapon potentials (number/year)	$2.95 \sim 4.49$	1.26 ~ 1.92

Table II: Plutonium Production and Weapon Potential



Depletion time

- VVER: 116 days
- Magnox: 1390 days

• Burnup

- VVER: 2900 MWd/tU
- Magnox: 695 MWd/tU

• Pu production

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• Grams of WG-Pu per MWd

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If North Korea decided to use the LWR as a source of weapon-grade plutonium for weapons, it could grow its nuclear weapons arsenal significantly by using a driver fuel/target system. Slated to be 100 MWth, or four to five times larger than the existing Yongbyon reactor, the LWR could produce roughly 20 kilograms of weapon-grade plutonium per year.¹⁷ At 3-4 kilograms of plutonium per weapon, twenty kilograms is enough for 5-6 nuclear weapons per year. The actual annual amount of weapon-grade plutonium could vary significantly, depending on the reactor's actual performance.

¹⁷ ISIS did not perform detailed calculations but a rough estimate was conducted with the support of a reactor expert who was familiar with the use of driver fuel/target systems in reactors. The estimate assumes a 70-80 percent capacity factor, a conversion of 0.85 grams of weapon-grade plutonium per megawatt-thermal-days and an estimated 10 percent reduction in plutonium output to account for the plutonium produced in the driver fuel, which is not usable. The resulting estimate is 19.5-22.3 kg weapon-grade plutonium per year. To assess enrichment requirements, the nuclear reactor expert said that a rule of thumb is that a core composed of 10-20 percent LEU driver fuel would have the same amount of uranium 235 as a core of 3.5 percent LEU fuel.

Figure 6. Institute for Science and International Security report^[7]



• *Seperated Pu

- VVER: 3.74 ~ 4.28 kg
- Magnox: 19.2 ~ 21.9 kg

• **Annular Pu production

- VVER: 11.8 ~ 13.5 kg/year
- Magnox: 5.04 ~ 5.75 kg/year

***Number of nuclear weapon potentials

- VVER: 2.95 ~ 4.49 #/year
- Magnox: 1.26 ~ 1.92 #/year

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Grams of WG-Pu per MWd (gPu/MWd)	0.512	0.876
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* Pu production × Capacity factor $(70 \sim 80 \%)$ × Reduction in output (10 %)

- ** Separated Pu \times 365 days \div Depletion time
- *** Annular Pu \div (3~4 kg Pu per weapon)



Plutonium production vs depletion time

- At one operation, the Magnox has a more Pu production than the VVER.
- During the same operation time, the VVER has a higher Pu production rate than the Magnox. (~130 % higher)



Figure 7. Comarison of plutonium production over depletion time by reactor types

Conclusions



- The construction of a new Experimental Light Water Reactor (ELWR) by North Korea continues to pose a significant nuclear proliferation threat.
- In this work, the plutonium production capacity for this ELWR was estimated, assuming it is based on the Russian VVER reactor design.
- It was estimated that North Korea's existing Magnox-type reactor can produce 1.26 to 1.92 nuclear weapons per year, while the ELWR can produce 2.95 to 4.49 nuclear weapons per year.
- This result undervalues from the estimation provided in the ISIS report.



	ELWR	5 MWe reactor
Reactor power (MWth)	100	25
Initial uranium mass (tons)	4	50
Grams of WG-Pu per MWd (gPu/MWd)	0.512	0.876
Number of nuclear weapon potentials (number/year)	2.95 ~ 4.49	1.26 ~ 1.92
ISIS's estimation (number/year)	5~6	

Conclusions



Pyongyang's desire for nuclear electricity with LWRs is likely genuine since it

has pursued acquisition of LWRs since 1985, first from the Soviet Union, then

from the United States, and now on its own. Though it is technically possible

unlikely because North Korea's existing gas-graphite reactor is more suitable

<Siegfried S. Hecker. (December 20, 2010). "Redefining denuclearization in North Korea", Bulletin of the Atomic Scientists>

that the LWR will be used to produce bomb-grade plutonium, I consider it

for the production of bomb materials than Pyongyang's LWR.

Limitations

Light Water Reactors (LWRs) are generally not well-suited for plutonium production.



Figrue 8. Pu production over burnup per initial uranium ton by reactor types^[8]

• Future works

- This work focused on point-depletion calculations and did not consider neutron leakage and operational periods.
- Incorporating 3D modeling will be needed to attain more accurate results, which may lower the estimated WG-plutonium production capacity.
- Accounting for reactor cooling periods will also be necessary.

Future work (Doing)

CT&RPL

• 3D Modeling

We are using Monte-Carlo code Serpent2 for full-core 3D analysis.

Fig. 2.1.1.5 Fig. 2.1.1.6

boron steel 7.51 g/cm³

Nominal density

7.86

2-D Fuel bundle



TABLE 2.1.1.1. WWER-440 DESIGN PARAMETERS



Core descriptio Fuel type: Enrichment w/o (given value) 1375 MW Geometry (including all supporting elements and enrichments of particular fuel rods) Rated thermal power Rated electrical power 440 MW Fig. 2.1.1.3 84.4 W/cm3 Specific power density Number of fuel rods Total mass of UO₂ per FA 136 kg (119.75 kg U metal) TABLE 2.1.1.1. (cont.) Pressure at core inlet 1225.83 N/cm Lattice pitch 1.22 cm Net core flow HFP inlet temperat 43000 m³/h Spacer grid: Material 269°C 300°C 302°C Control Assembly HFP average core outlet temperature HFP average fuel cladding temperature 12X18H10T stainless-steel 7.86 g/cm³ Fig. 2.1.1.4 707°C in radial plane (cross section of control absorber) Number, location, axial dimensions, mass fractions of materials in one spacer grid nsions, mass or in axial p Core: Total fuel loading in the core 47 000 kg UO2 0.118 kg Mass of one spacer grid 42 000 kg U m Fig. 2.1.1.1 Density Geometry Number of fuel batches for initial core Z110 - 1%Nb Material Number of add type of fuel assemblies in each batch Enrichment of each type of fuel assembly Loading pattern for initial core showing position 114, 133, 102 Zirconium a 6.52 g/cm³ 0.515 cm 1.6%; 2.4%; 3 Density Outer radius Fig. 2.1.2.1 144 cm of each type fuel assembly Thickness 0.075 cm Effective core radius Location in con in the center of FA Reload pattern Location of control assemblies Fig. 2.1.1.2 Fuel rod 10 18 1.5 -9.8 17.7 1.5 1.9 70.5 Identification of control assemblies or rod groups used for control and shut down. 69.2 6 Z110 - 1%Nb zirconium alloy 6.52 g/cm³ 0.455 cm 0.386 cm Density Outer radius Inner radius Fig. 2.1.1.1 eight per cen Water temperati Water pressure 1225.83 N/cm² Nb daterial of core basket (thermal shield) 08X18H10T Pellet: Material Thickness of core basket UO2 0.95x10.2 g/cm³ 0.3800±0.0025 cm 98.97 1.0 0.03 97.47 2.5 0.03 Z110 Z125 08X18H10T cification of other components (core barrel) Percent of theo tical density x density Outer Radius Average mass of peller 13.17 g UO₂ (11.59 g metal) olv data The following information is valid for each type of fuel assembly (FA): Height Number in fuel rod 3cm 82 14.7 cm 0.3 cm Zirconium allo (Z125- 2.5%N Fuel assembly nitch Height of UO₂ (HFP) 246 cm Gap between Shell material ~ 0.15 MPa Initial He pressure, if any Mass of UO, per fuel rod in a fuel assembly 1080 g of UO₂ r 1035 g Shell thickness 0.2 cm

TABLE 2.1.1.1. (cont.)

3-D Whole core



Figure 9. VVER-440 Design parameters^[9]

Future work (Doing)

• 3D Modeling

- We aim for the similar R/H of the active core and linear heat densities.
- Rod number per bundle, rod diameter, rod pitch, and bundle pitch are same.
- To achieve the similar R/H, the number of fuel bundle is determined while the same number of fuel rods per bundle.
- Considering the similar R/H and linear heat density simultaneously, the active core height are decided.



Figure 10. Core configurations of VVER-440^[9] and ELWR (Left: VVER-440 / Right: ELWR)



Table III: Design parameters of the VVER-440 and ELWR

	VVER-440	ELWR
Thermal power (MWt)	1375	100
Initial uranium mass (tons)	~ 37	3.93
Rod number per bundle	126	126
Number of fuel bundles	312	60
Active core radius (cm)	137	60
Active core height (cm)	246	110
Radius/Height of core	0.555	0.548
Linear heat density (W/cm)	142	120

Future work (Doing)

CT&RPL

• 0-D vs 3-D calculations

- Due to the more computational cost of the 3-D calculation compared to 0-D depletion calculation, the 3-D calculation was performed with a longer depletion time step.
- The 3-D calculations result in a smaller amount of plutonium (Pu) production, with approximately a 15.3% difference compared to the 0-D calculations.



Figure 11. Comarison of plutonium production over depletion time by calculation types





Thank you for listening