

Preliminary Analysis of Boron-free Operation Small Modular Reactor using McCARD and DeCART2D



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01 Introduction



01 Introduction

- **Background**

- *Boron-free Operation Small Modular Reactors*, such as the innovative SMR (**i-SMR**), aims for extended operational cycle, and flexible thermal power range and Soluble Boron-free (SBF) conditions.
- SBF conditions enhances system integrity by reducing liquid waste, simplifying chemical control, and lowering the risk of boron dilution accidents.
- To achieve SBF conditions, different operating conditions (e.g. enriched gadolinium) than commercial PWRs are required. This means that a core analysis code system suitable for SBF conditions is required.
- To establish a new high-fidelity nuclear core analysis code system for SBF conditions, the errors in an existing core design analysis code system must be quantitatively evaluated.



01 Introduction

- **Introduction**

- *Target SBF system*

Our goal is to perform and quantify the errors for newly-developing i-SMR, but the specifications of i-SMR have not been finalized. Accordingly, old data from the paper of J.S. Kim^{[1][2]} was used in this study.

- *Target core design code*

Deterministic 2-Step procedure core analysis code system is being performed utilizing two libraries for the target SBF system by *DeCART2D/MASTER*, which are developed by KAERI.

- *Reference*

- ✓ *McCARD* Monte-Carlo (MC) calculations were performed to provide the reference solutions for V&V calculations.
- ✓ Based on the results for the target SBF system, the error was evaluated through comparison and verification in the *McCARD* and *MASTER* codes.



01 Introduction

- **Computational Code**

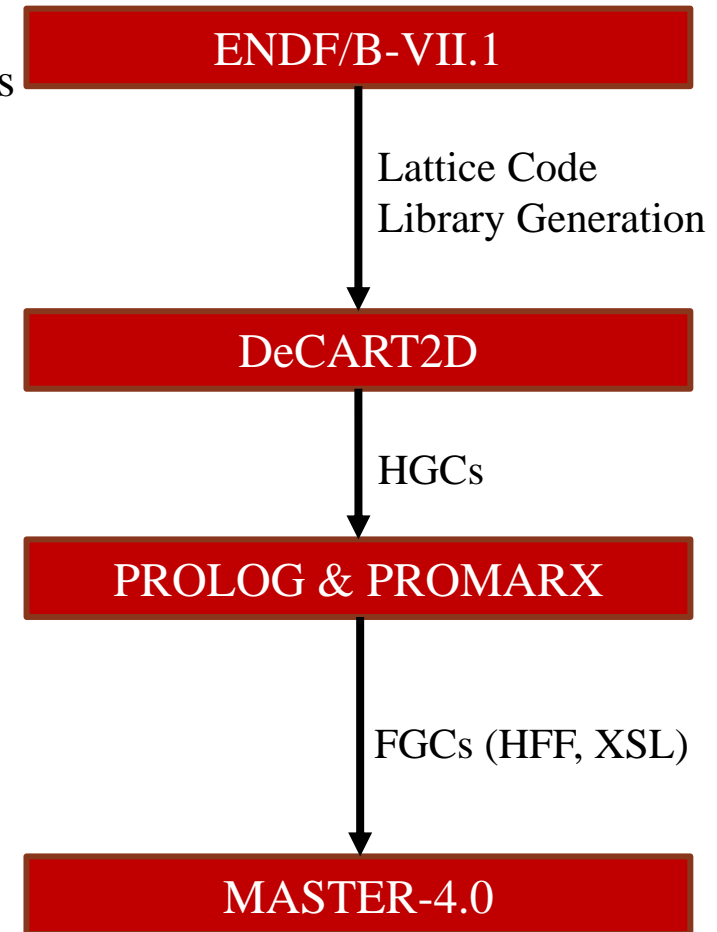
- **McCARD^[3]**

- ✓ Developed by Seoul National University
- ✓ Neutron/Photon Monte-Carlo transport analysis
- ✓ Can handle Continuous cross-section library
- ✓ Used ENDF/B-VII.1
- ✓ Used as design code for *JRTR* and *KJRR*

- **DeCART2D^[4]/MASTER^[5]**

- ✓ Developed by KAERI
- ✓ 2-Step procedure core analysis code system
 - DeCART2D: MOC Calc. (Lattice code)
 - MASTER-4.0: Nodal Calc. (Whole Core analysis code)
- ✓ ENDF/B-VII.1 based on Library (47-Groups)
- ✓ Used as design analysis for *SMART*, *ARA Rx*

DeCART2D/MASTER 2-Step procedure code system



02 Design & Specifications



02 Design & Specifications

- **Core Specifications of Boron-free Operation SMR**
 - Extended operational cycle (**24 months**) + *SBF Condition*
 - ✓ Need a lot of burnable absorber for excess reactivity control.
 - ✓ The stainless steel was used as a reflector.

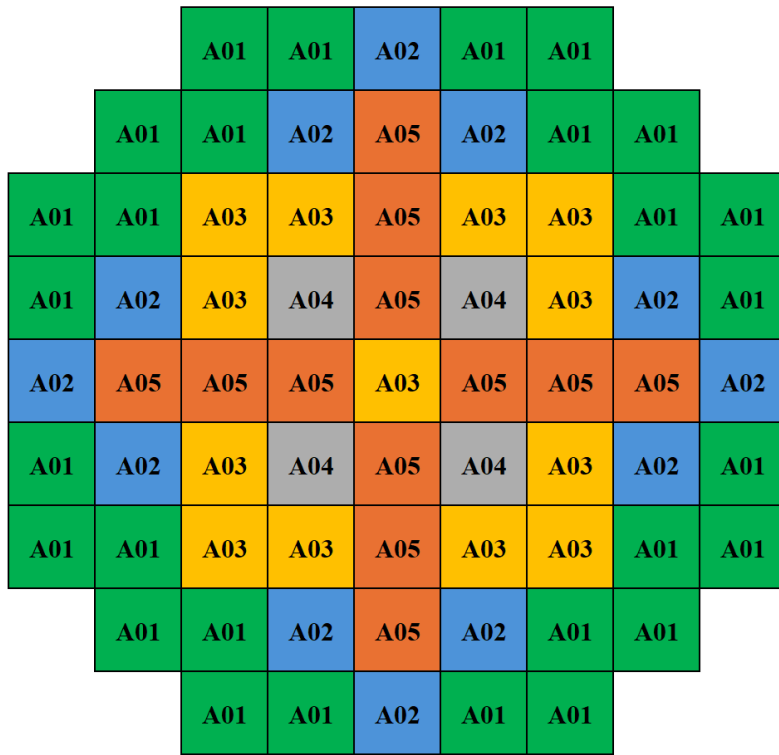


Fig. 1. Configuration of the core models

Table. I. Core Specifications

Parameters	Value
Power	540 MW _{th}
Number of FA	69
Refueling cycle	24 months
Batch	2 batches
Inlet coolant	548.15 K
Outlet coolant	598.15 K
Boron concentration	0 ppm
Active core height	240 cm

02 Design & Specifications

- **Enriched gadolinium in Burnable Absorber**
 - Excess reactivity control
 - ✓ Commercial PWR → Soluble Boron + Burnable Absorber Rods + Control Rods
 - ✓ Boron-free SMR → Burnable Absorber Rods + Control Rods
 - Enriched gadolinium are required due to the thermal conductivity and content of UO_2
 - The combined abundance of ^{155}Gd and ^{157}Gd , which have high absorption cross-sections, was enriched to 50% and 70%, respectively.

Table. II. Abundance of natural and enriched Gadolinium isotopes

Isotope	Abundance (%)		
^{152}Gd	0.20	0.14	0.09
^{154}Gd	2.18	1.57	0.94
^{155}Gd	14.80	24.30	34.02
^{156}Gd	20.47	14.72	8.83
^{157}Gd	15.65	25.70	35.98
^{158}Gd	24.84	17.86	10.71
^{160}Gd	21.86	15.72	9.43
Sum ($=^{155}\text{Gd} + ^{157}\text{Gd}$)	30.45 (Nat.)	50.00	70.00

02 Design & Specifications

FA Specifications of Boron-free Operation SMR

- FA Type (Westing House) → Control Rod worth
- Additionally, high-concentration, low-content, gadolinia was also loaded.

Table III. FA Specifications

Parameters	Value
Array	17 x 17
Rod pitch	1.26 cm
Assembly pitch	21.5 cm
Pellet radius	0.4096 cm
Guide tube Inner / Outer radius	0.5613 / 0.6124 cm

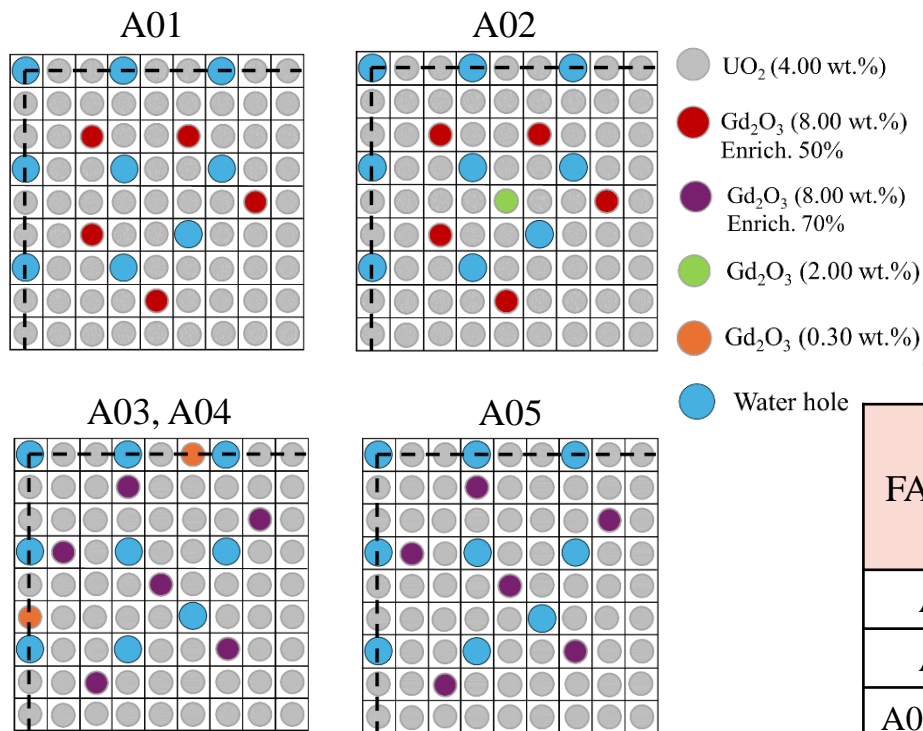


Table IV. Composition of Burnable Absorber by FA Type

FA Type	Gd atomic ratio (a/o)	Gd ₂ O ₃ (wt.%)		U ₂₃₅ Enrichment (wt.%)	
		High	Low	High	Low
A01	50	8.00	-	2.30	-
A02	50	8.00	2.00	2.30	2.90
A03, A04	70	8.00	0.30	2.30	3.90
A05	70	8.00	-	2.30	-

Fig. 2. Configuration of the FA models

02 Design & Specifications

- **Improvement of Library Generation Code System for DeCART2D**

- DeCART2D cross-section library for lattice code contains nuclide-wise nuclear reaction cross section and burnup dependent data.
- Recently, Kim and Park applied the new cross section library correction procedure into the KAERI library generation code system.
 - ✓ **CORRXS** : Multi-group Cross Section Correction
 - ✓ **CORRIT** : Resonance Integral Table Correction

- **Correction Cross-section**

- Correction factors based on MC
 - ✓ Absorption, nu-fission cross section
 - ✓ Scattering Matrix

$$f_{x,g}^{n+1} = \frac{\sigma_{x,g}^{MC} \phi_{x,g}^{MC}}{\sigma_{x,g}^{DE,n} \phi_{x,g}^{DE,n}}$$

$$\sigma_{x,g}^{DE,n+1} = f_{x,g}^{n+1} \sigma_{x,g}^{DE,n}$$

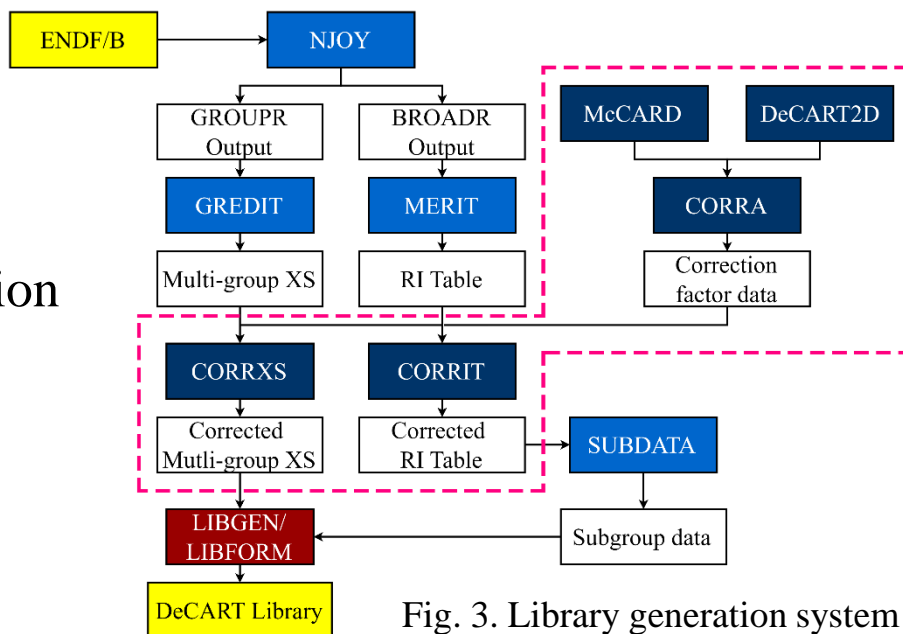


Fig. 3. Library generation system

※ $\sigma_{x,g}^{MC}$ and $\sigma_{x,g}^{DE,n}$: g^{th} energy group cross section for reaction type x by McCARD and DeCART2D.
 ※ $\phi_{x,g}^{MC}$ and $\phi_{x,g}^{DE,n}$: g^{th} energy group flux produced by McCARD and DeCART2D.

02 Design & Specifications

- **Calculation Conditions**
 - Based on ENDF/B-VII.1 nuclear data library
 - Temperature condition: HFP
 - ✓ Fuel: 900 K, Clad: 600 K, Moderator: 600 K
 - Boron concentration: 0 ppm (SBF)
- **McCARD calculation conditions**

Case	Histories / Active cycles / Inactive cycles	Stochastic uncertainties ($=1\sigma$)
FA	50000 / 500 / 200	< 14 pcm
2D core	50000 / 1500 / 1000	< 13 pcm
Depletion	20000 / 150 / 100	< 40 pcm

- **Cross-Section Library for DeCART2D**

Library name	Library code	Note
<i>Old Library</i>	PV01-CR08	Existing library for analysis commercial PWR - U^{235} , U^{238} corrected
<i>New Library</i>	PV05-iSMR-CR04	Preliminary library for analysis Boron-free SMR - U^{235} , U^{238} , Gd^{154} , Gd^{155} , Gd^{156} , Gd^{157} , Gd^{158} , Gd^{160} , H^1 corrected



03 Numerical Results



03 Numerical Results

- **Boron-free Operation SMR Optimized Library Generation**
 - Large errors in group-wise reaction rates from uncorrected DeCART2D library
 - New DeCART2D library generation by multi-round corrections (improved system)
 - Corrected U^{235} , U^{238} , Gd^{154} , Gd^{155} , Gd^{156} , Gd^{157} , Gd^{158} , Gd^{160} , H^1 isotopes

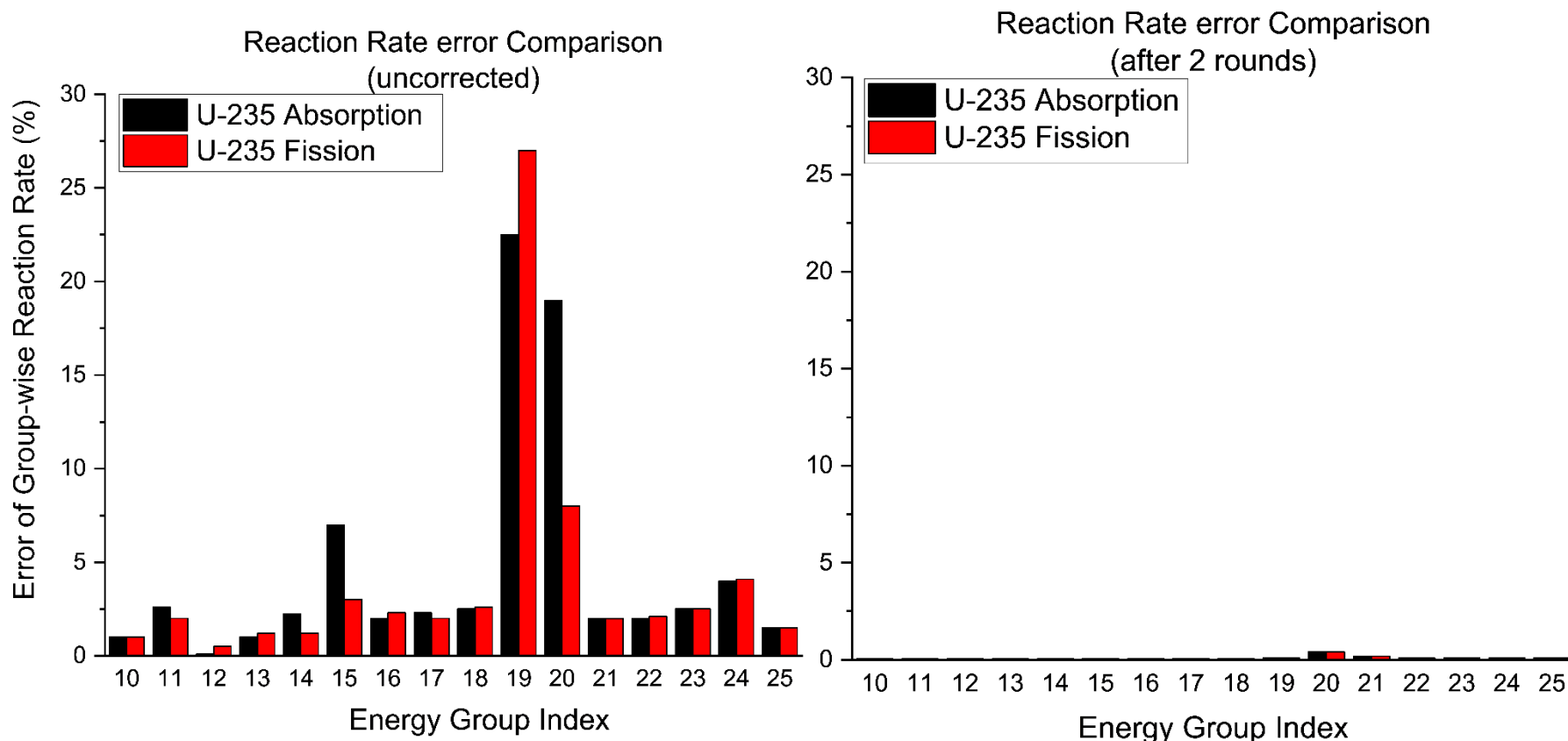


Fig. 4. Example of multi-round group-wise reaction rate corrections

03 Numerical Results

• FA Benchmark

- 5 FA benchmark problems - DeCART2D and McCARD (*reference*).
 - ✓ Old Library : Uncertainties in k_{eff} are less than 150 pcm.
 - ✓ New Library : Uncertainties in k_{eff} are less than 50 pcm. → improvement of accuracy

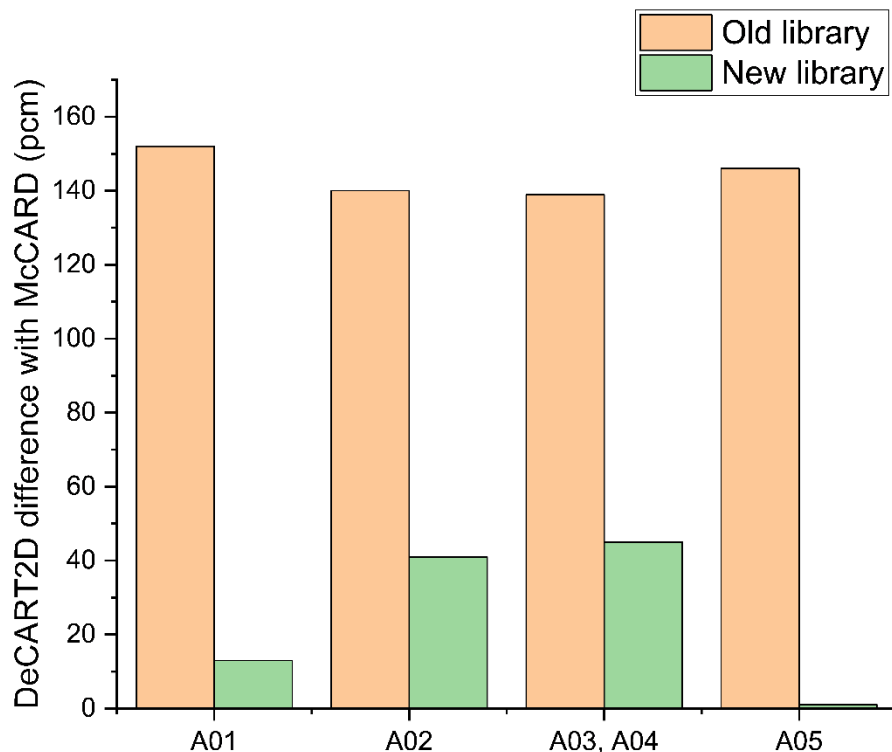


Fig. 5. Results of the FA models

Table. V. Multiplication factors of FA

Case	McCARD ¹⁾	DeCART2D difference with McCARD (pcm)	
	k_{eff}	Old ²⁾	New ³⁾
A01	1.09874	152	13
A02	1.06743	140	41
A03, A04	1.01797	139	45
A05	1.03422	146	1

1) Stochastic uncertainty ($=1\sigma$) is less than 0.00014

2) PV01-CR08 library

3) PV05-iSMR-CR04 library

4) $k_{DeCART2D} > k_{McCARD}$



03 Numerical Results

- **2D Core Benchmark**

- 2D core benchmark problem for the SBF system were performed by DeCART2D and McCARD.
- ✓ Old Library : Uncertainties in k_{eff} are about 400 pcm.
- ✓ New Library : Uncertainties in k_{eff} are about 20 pcm. → improvement of accuracy

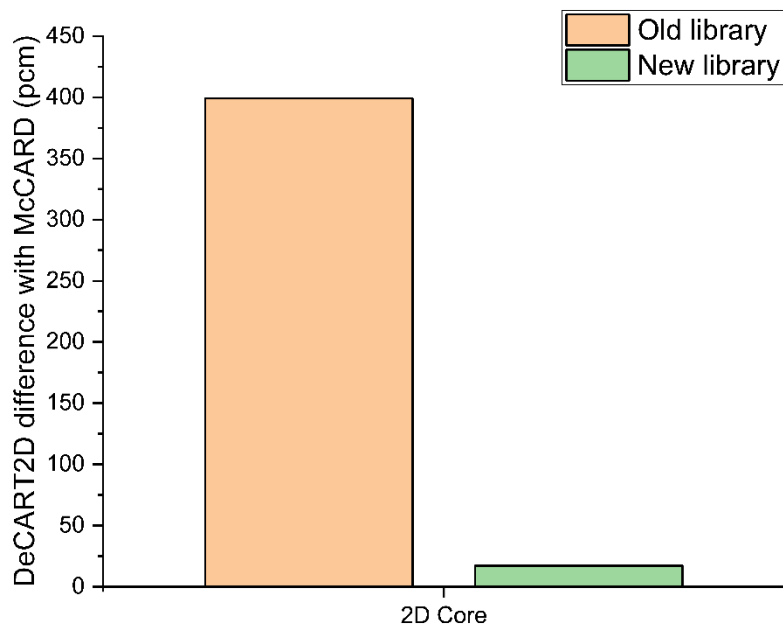


Fig. 6. Result of the 2D Core

Table. VI. Multiplication factors of 2D Core

Case	McCARD ¹⁾	DeCART2D difference with McCARD (pcm)	
	k_{eff}	Old ²⁾	New ³⁾
2D Core	1.04240	399	17

1) Stochastic uncertainty ($=1\sigma$) is less than 0.00013

2) PV01-CR08 library

3) PV05-iSMR-CR04 library

4) $k_{DeCART2D} > k_{McCARD}$



03 Numerical Results

- **2D Core Benchmark**

- RMSE (maximum error) in FA-wise power distribution

- ✓ Old Library : 1.08% (1.89%)
- ✓ New Library : 0.47% (0.90%)

	E	F	G	H	J	
	A03	A05	A05	A05	A02	Assembly
5	-0.52%	-0.76%	-0.97%	-0.59%	1.19%	Old ¹⁾
	-0.27%	-0.55%	-0.61%	-0.28%	0.42%	New ²⁾
	A05	A04	A03	A02	A01	
6	-0.76%	-0.74%	-0.94%	-0.37%	1.30%	RMSE
	-0.55%	-0.41%	-0.44%	0.00%	0.44%	1.08%
	A05	A03	A03	A01	A01	0.47%
7	-0.97%	-0.94%	-0.36%	0.41%	1.89%	Max.
	-0.61%	-0.44%	0.17%	0.42%	0.57%	1.89%
	A05	A02	A01	A01		0.90%
8	-0.59%	-0.37%	0.41%	1.74%		
	-0.28%	0.00%	0.42%	0.90%		
	A02	A01	A01			
9	1.19%	1.30%	1.89%			
	0.42%	0.44%	0.57%			

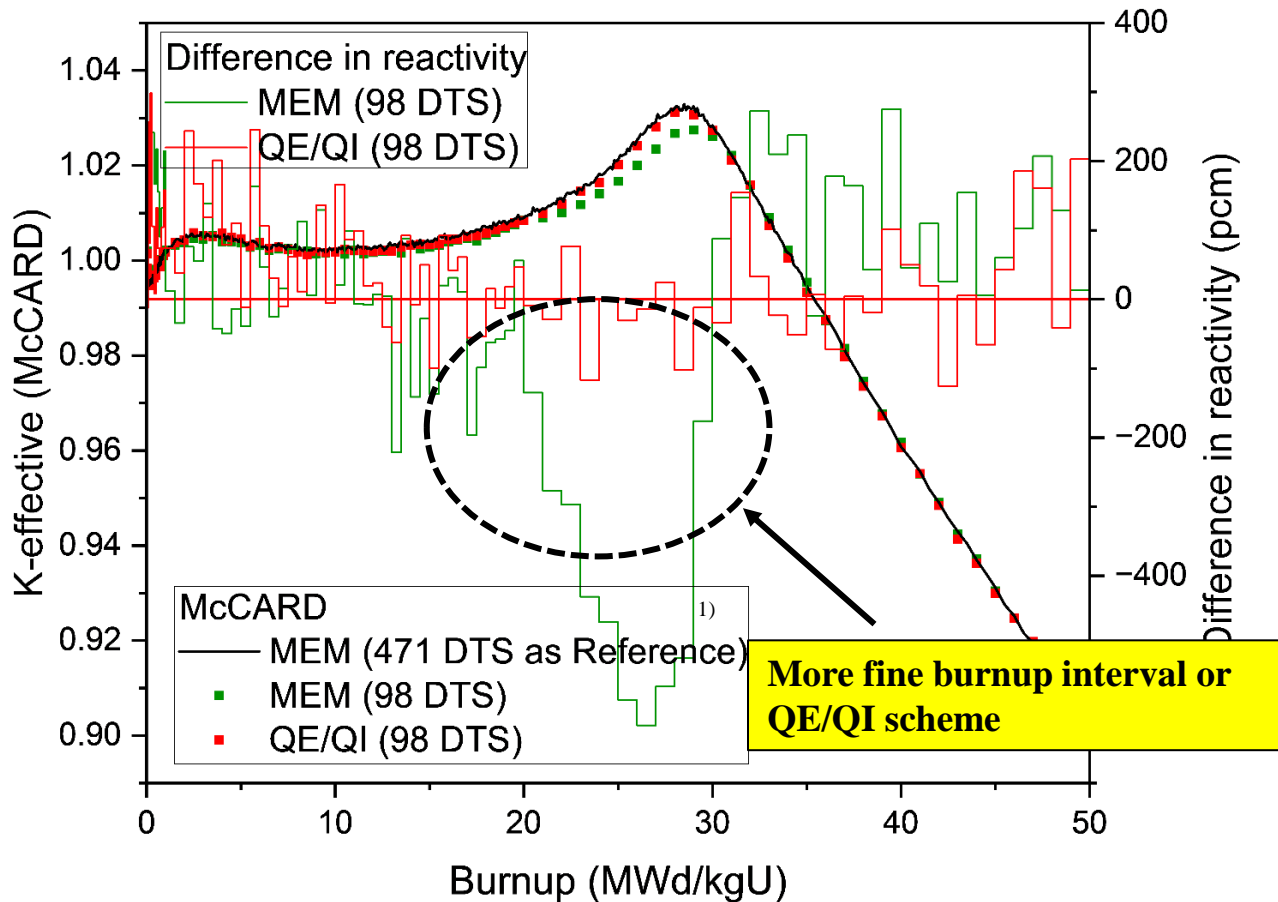
Fig. 6. FA-wise power distribution by McCARD and DeCART2D

1) PV01-CR08 library
 2) PV05-iSMR-CR04 library



03 Numerical Results

- **Reference Solution Generation for FA Depletion Problem**
 - A03-type FA problem.
 - McCARD reference solutions : 98 DTS with QE/QI option.



Burnup (MWd/kgU)	Difference (pcm)	
	MEM (98 DTS)	QE/QI (98 DTS)
0	61	-55
1	65	24
2	-115	-63
5	-110	-10
10	-95	-95
15	-111	-55
20	-50	-57
25	-430	-91
30	-219	-99
40	109	-17
50	-81	56
RMSE	160	83

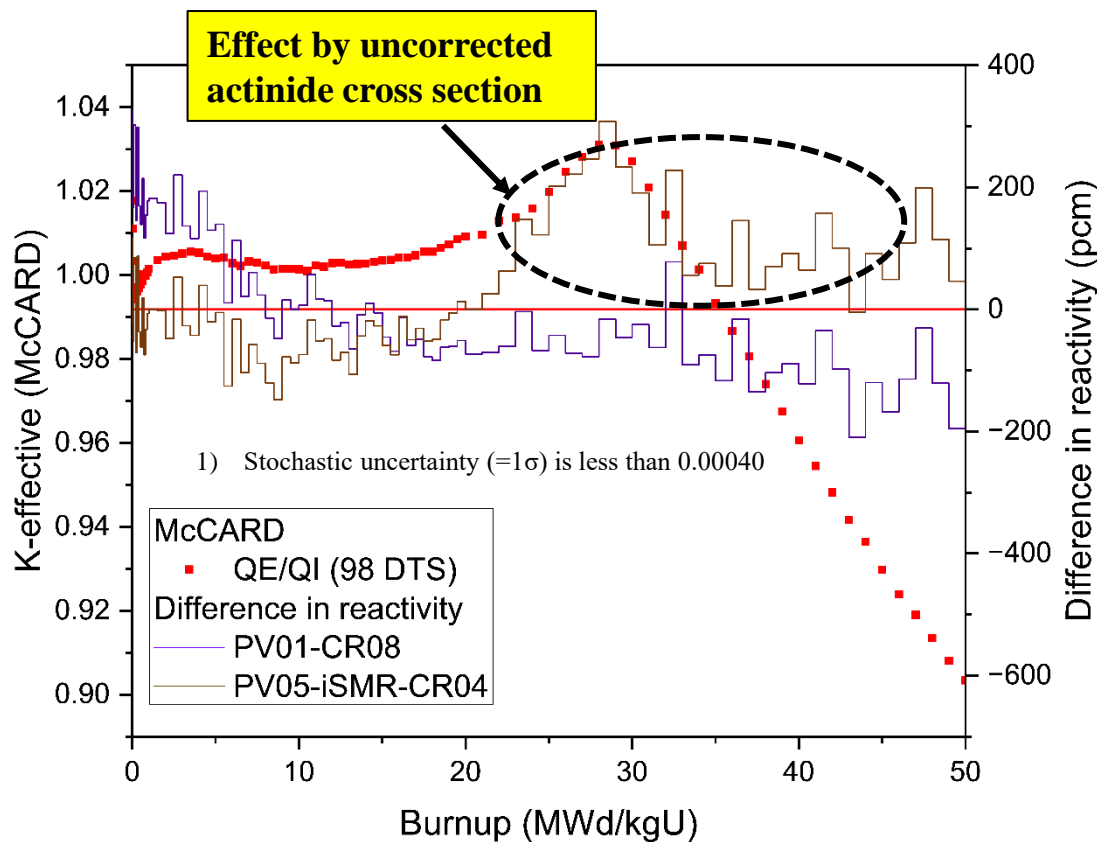
Fig. 7. Differences in reactivity among McCARD burnup analyses with various options



03 Numerical Results

FA Depletion Problem

- RMSE(New Library) = 98 pcm < RMSE(Old Library) = 130 pcm
- Errors in reactivity after MOC may induce from the uncorrected actinide isotopes



Burnup (MWd/kgU)	Difference (pcm)	
	Old	New
0	180	-5
1	182	0
2	124	47
5	139	5
10	7	-73
15	-45	-59
20	-72	0
25	-42	201
30	-24	191
40	-121	64
50	-63	185
RMSE	130	98

Fig. 8. Differences in reactivity among McCARD and DeCART2D

03 Numerical Results

- **3D Core calculation**
 - Compared KARMA/ASTRA results of the paper of J.S. Kim^{[1][2]}
 - Similar *ASI* and *F_q*, but Large difference in *reactivity* between ASTRA and MASTER
 - ✓ Uncertain and unclear specification : cutback region, reflector, enrichments

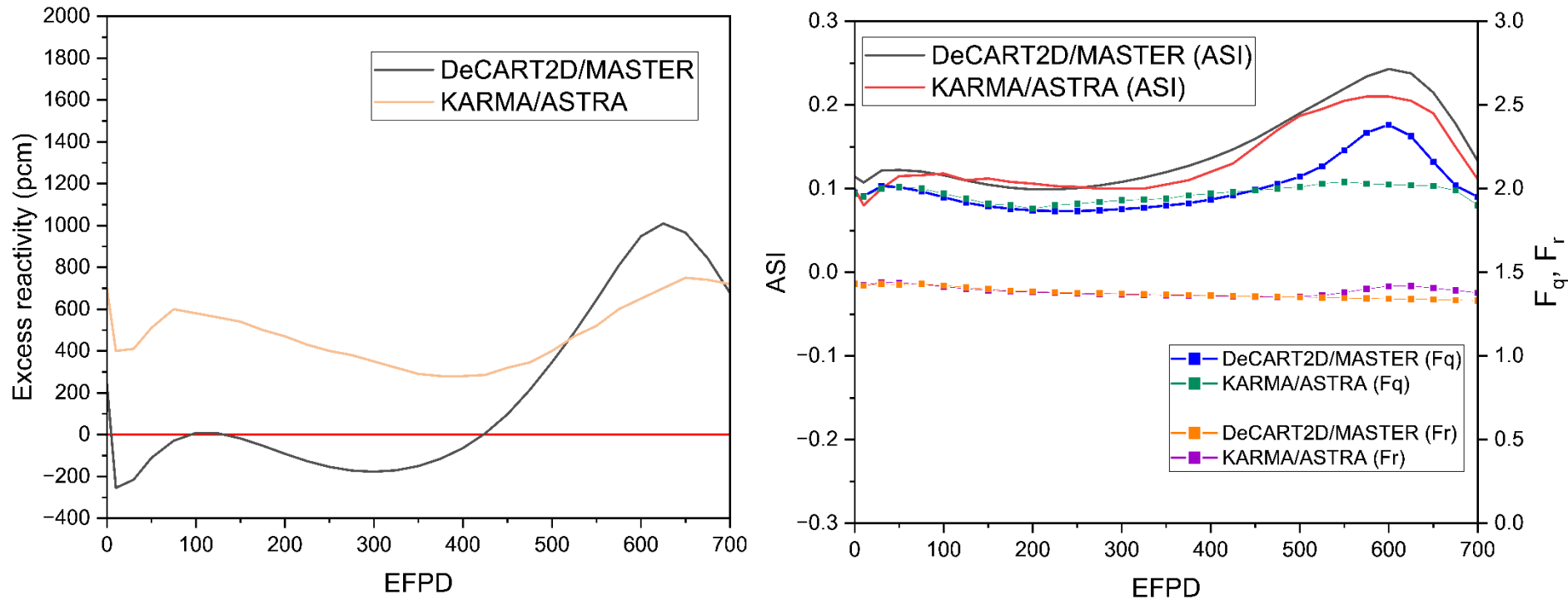


Fig. 9. Differences DeCART2D/MASTER and KARMA/ASTRA

04 Conclusions & Future works



04 Conclusions & Future works

- **Conclusions**

- Preliminary analysis for Boron-free SMR has been successfully conducted using *DeCART2D/MASTER* 2-Step procedure code system.
- Reference solutions were generated by *McCARD* MC code.
- Generated new Boron-free SMR optimized DeCART2D library (*New library*) using the improved KAERI library generation code system.

- Summary of Results

Case	ρ difference	
	Old	New
A01	152	13
A02	140	41
A03, A04	139	45
A05	146	1
2D Core	399	17

Power distribution	Difference (%)	
	Old	New
RMSE	1.08	0.47

Burnup	Difference (pcm)	
	Old	New
RMSE	130	98

- The new Boron-free SMR optimized library shows better performance in estimating reactivity, power distributions.



04 Conclusions & Future works

- **Future works**
 - In the near future, 3D Core multi-cycle analysis for up-to-date or finalized i-SMR core design by MASTER code will be conducted.
 - ✓ Core follow analysis
 - ✓ Comparison of nuclear design parameters
 - The i-SMR optimized library will be generated, especially the cross sections for Pu and minor actinide isotopes will be corrected for the depletion analysis.



05 References

- [1] J. S. Kim, “Reactor core design with enriched gadolinia burnable absorbers for soluble Boron-Free operation in the innovative SMR”, Nuclear Engineering and Design, 2024.
- [2] J. S. Kim, et al. "Applicability Evaluation of Enriched Gadolinium as a Burnable Absorber in Assembly Level for Boron-Free iSMR." In Transactions of the Korean Nuclear Society Spring Meeting, Jeju, Korea, May 19-20, 2022.
- [3] H. J. Shim et al., “McCARD: Monte Carlo Code for Advanced Reactor Design and Analysis,” *Nucl. Eng. Technol.*, 44, pp.151-176, 2012.
- [4] J. Y. Cho, et al., “DeCART2D v1.1 User’s Manual,” KAERI/UM-40/2016, 2016.
- [5] J. Y. Cho, et al., “MASTER v4.0 User’s Manual,” KAERI/UM-41/2016, 2016
- [6] C. H. Kim, et al. "Preliminary Benchmarking of DeCART2D/MASTER Two-Step Core Design System for APR-1400 Benchmark using Improved Cross Section Library." Korean Nuclear Society Spring Meeting, Jeju, Korea, May 19-20, 2022.
- [7] H. J. Park, et al. “Monte Carlo Burnup and Its Uncertainty Propagation Analyses for VERA Depletion Benchmarks by McCARD,” *Nucl. Eng. Technol.*, 50, pp. 1043-1050, 2018.



Thank you!

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